

Theoretical analysis and experimental validation in DIII-D of predictive modeling for tungsten erosion and redeposition in tokamak divertors

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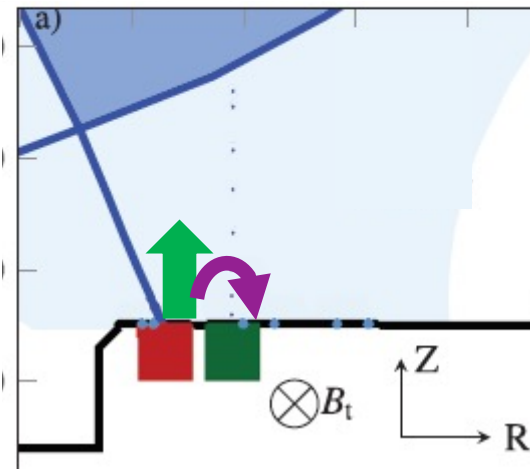
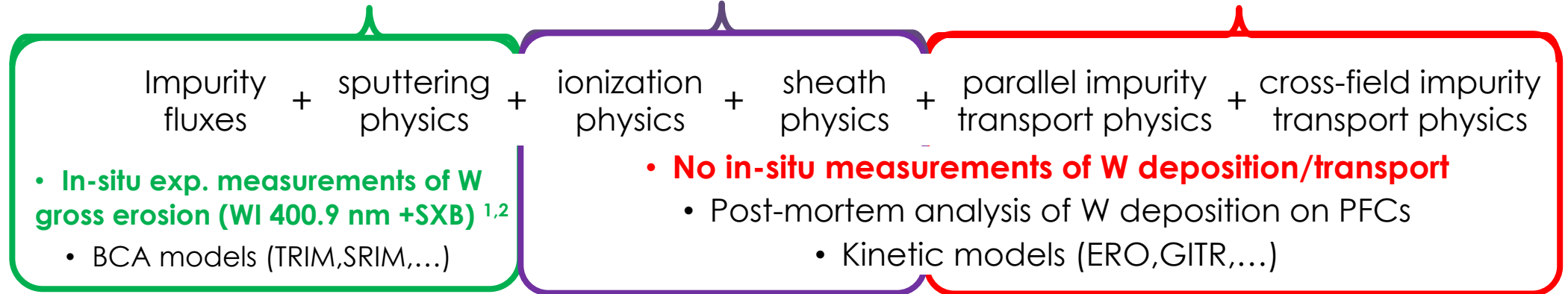
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Properties of W and Hydrogen in Edge Plasma of Fusion Devices*

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W net erosion from divertor PFCs largely determined by W prompt redeposition but no direct measurement available in tokamak divertors

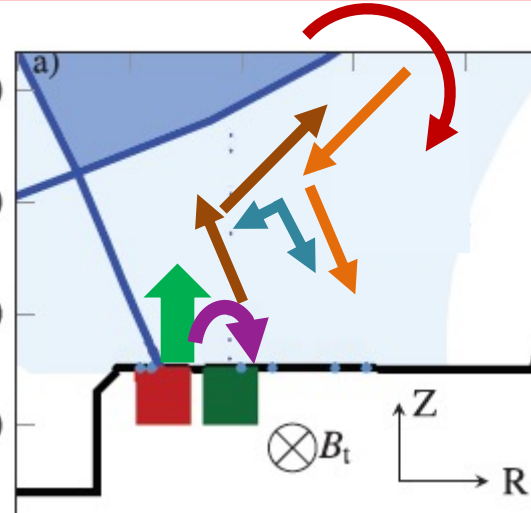
W net erosion in divertor \approx **gross erosion** \times (1 - prompt redeposition) \times (1 - non-prompt local redeposition)



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- | Impurity fluxes + sputtering physics | ionization physics + sheath physics | parallel impurity transport physics + cross-field impurity transport physics |
|--|--|--|
| <ul style="list-style-type: none">• In-situ exp. measurements of W gross erosion (WI 400.9 nm +SXB)^{1,2}• BCA models (TRIM,SRIM,...) | <ul style="list-style-type: none">• No in-situ measurements of W deposition/transport• Post-mortem analysis of W deposition on PFCs• Kinetic models (ERO,GITR,...) | |



- For impurity in trace approximation (concentration $\lesssim 1\%$) like W, non-prompt local redeposition driven by balance between:
 1. Friction with plasma particles (push particles toward divertor)
 2. Thermal forces (asymmetry in collisions due to temperature gradient take particles away from divertor)
 3. // electric field ($E \times B$ radial and poloidal drifts) \Rightarrow **Recirculation of impurity in divertor¹**

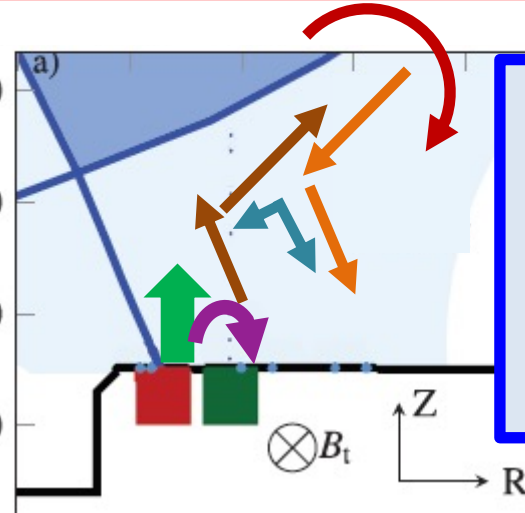


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$$W \text{ net erosion in divertor} \approx \text{gross erosion} \times (1 - \text{prompt redeposition}) \times (1 - \text{non-prompt local redeposition})$$

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- W prompt redeposition is usually large in attached plasma conditions and determines the overall W net erosion (prompt redeposition~1)

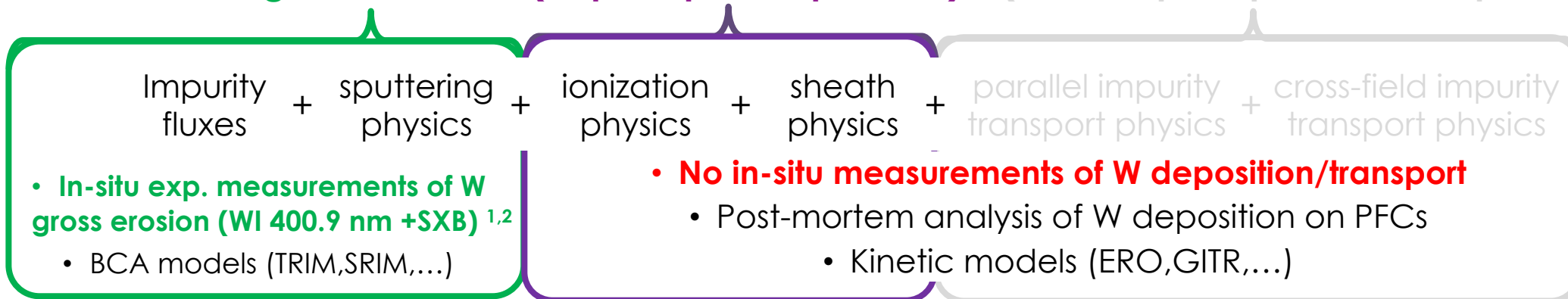


Worst case scenario:
 All W non-promptly redeposited transported upstream
 =
 W influx in SOL
 $W \text{ net erosion in divertor} \leq \text{gross erosion} \times (1 - \text{prompt redeposition})$



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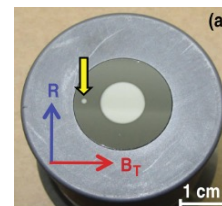
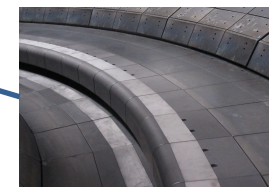
- Developing predictive models for W prompt redeposition in divertors requires to:

- understand fundamental physics processes governing W prompt redeposition
- validate predictive models of W prompt redeposition against experiments

Predictive model benchmarking

Metal ring experiment²

DIII-D experiments



small/large dots experiment¹

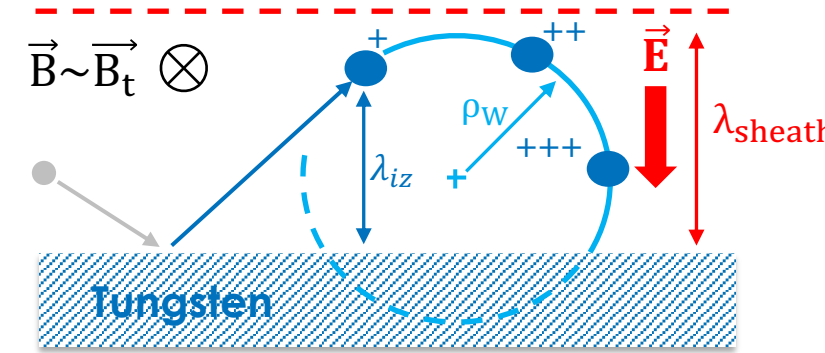
¹ D. Rudakov *Physica Scripta* 2014

² J. Guterl *Nuclear Fusion* 2019



Modeling of W prompt redeposition in presence of grazing magnetic field must include sheath effects and multiple ionizations of W

- Ionization of sputtered W near divertor surface due to collisions with electrons*
- **W prompt redeposition (redeposition of charged W impurities during their first gyration) affected by multiple ionizations of W impurities & electric sheath**

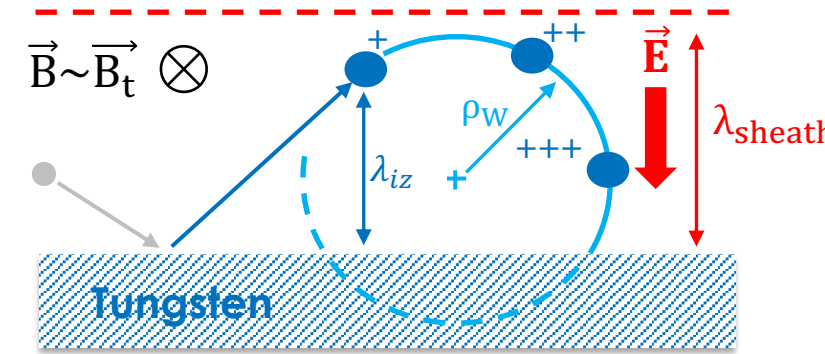


* could also be induced by CX (see David Tskhakaya's talk)



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- **W prompt redeposition (redeposition of charged W impurities during their first gyration) affected by multiple ionizations of W impurities & electric sheath**
- 3D trajectory of W impurities modeled by Monte-Carlo particle pusher with E and B fields embedded in ERO-D3D (HPC version of ERO¹)
 - Collisions of W impurities with plasma ion and neutral species negligible for W prompt redeposition in attached plasma conditions* ($\tau_{\text{collision}}\omega_c \gg 1$)
- **Effects of electric sheath and multiple W ionizations on W prompt redeposition qualitatively described by Brooks² and Fussmann³ in 90's ...**
- **...but quantitative modeling of W prompt redeposition now required for ITER W divertor and beyond!**

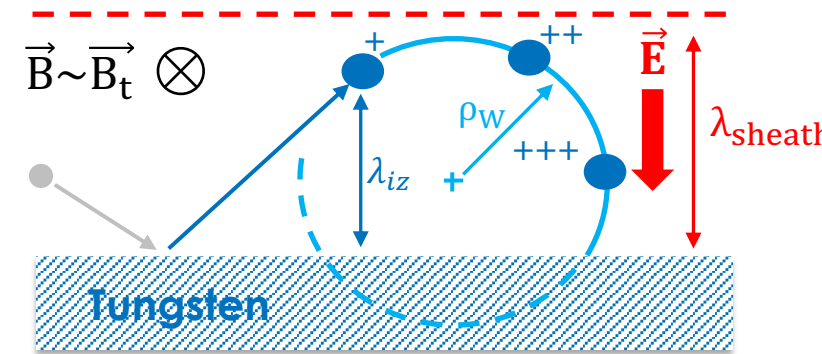
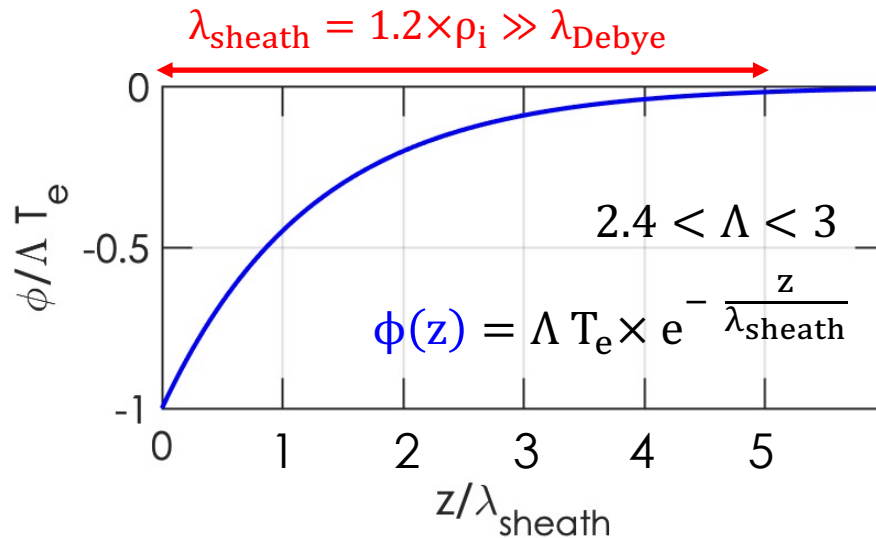


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W impurities ionized within the sheath because of the large sheath width due to magnetic field lines intersecting divertor targets at grazing incidence

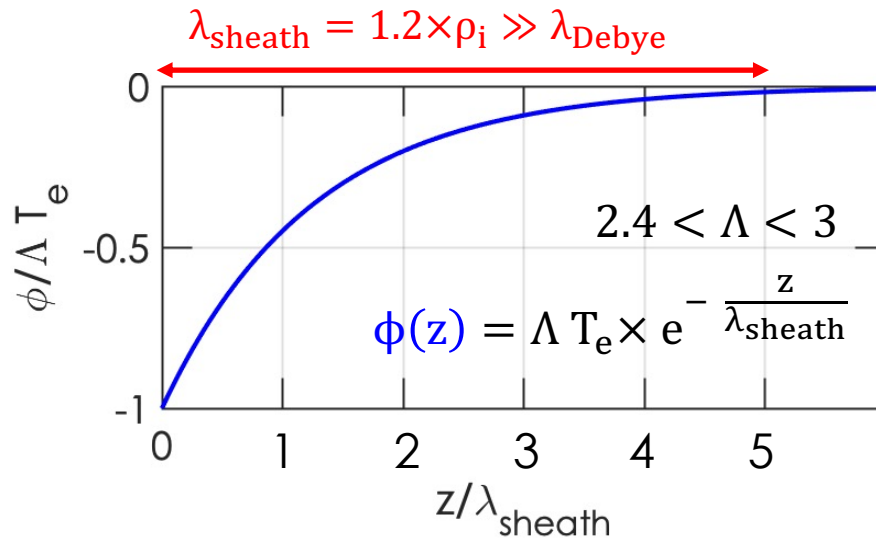
- Wide electric sheath (Chodura sheath) due to grazing magnetic field ($< 5^\circ$) in divertor¹: $\lambda_{\text{sheath}} \sim \rho_i$
- Sheath “frozen” for tungsten impurities (trace approximation)
- Sheath electric potential profile provided by kinetic simulations^{2,3}



¹ [D. Ryutov CPP 1996](#) ² [D. Coulette PPCF 2016](#) ³ [D. Tskhakaya JNM 2015](#)

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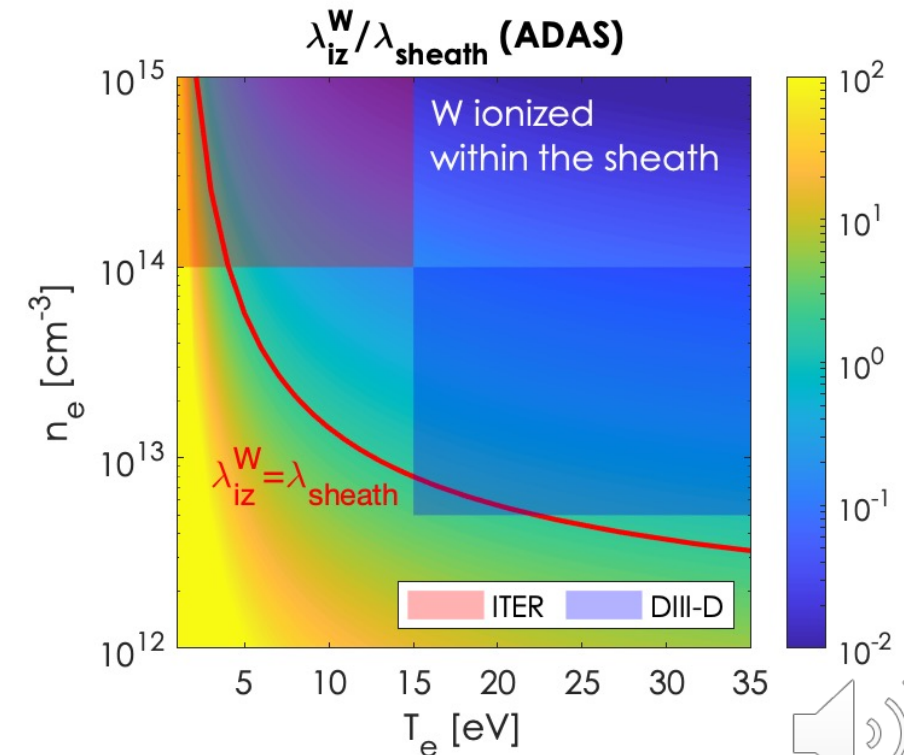
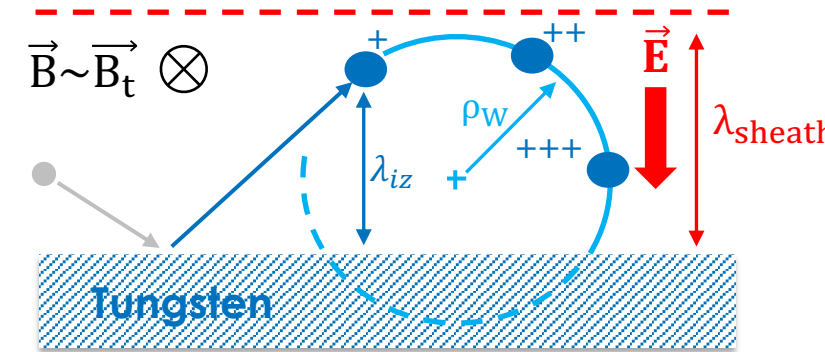
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- In ITER and DIII-D:

→ Sputtered neutral W ionized within the sheath :

$$\lambda_{iz} \lesssim \lambda_{\text{sheath}} \lesssim \rho_w$$



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W prompt redeposition strongly enhanced by the sheath electric field because of the large inertia of W impurity

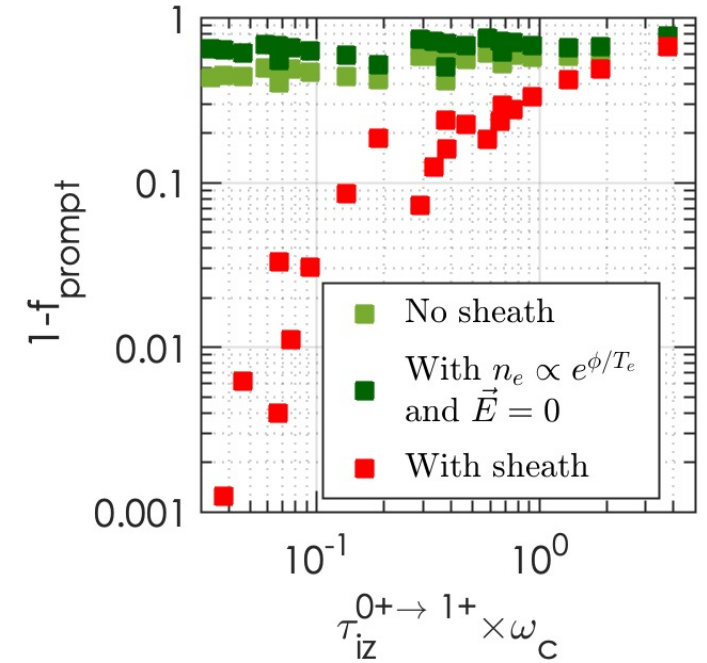
- When W ionized within the sheath ($\lambda_{iz} \lesssim \lambda_{\text{sheath}}$), W prompt redeposition affected by Chodura sheath due to:

- increase of λ_{iz} due to the decay of n_e in the sheath

$$n_e(z) = n_{e,0} e^{\phi(z)/T_e}$$

- acceleration of impurity toward material surface by the sheath electric field

- Sheath electric field strongly enhances W prompt redeposition and has stronger effects than multiple W ionizations and decay of n_e in the sheath



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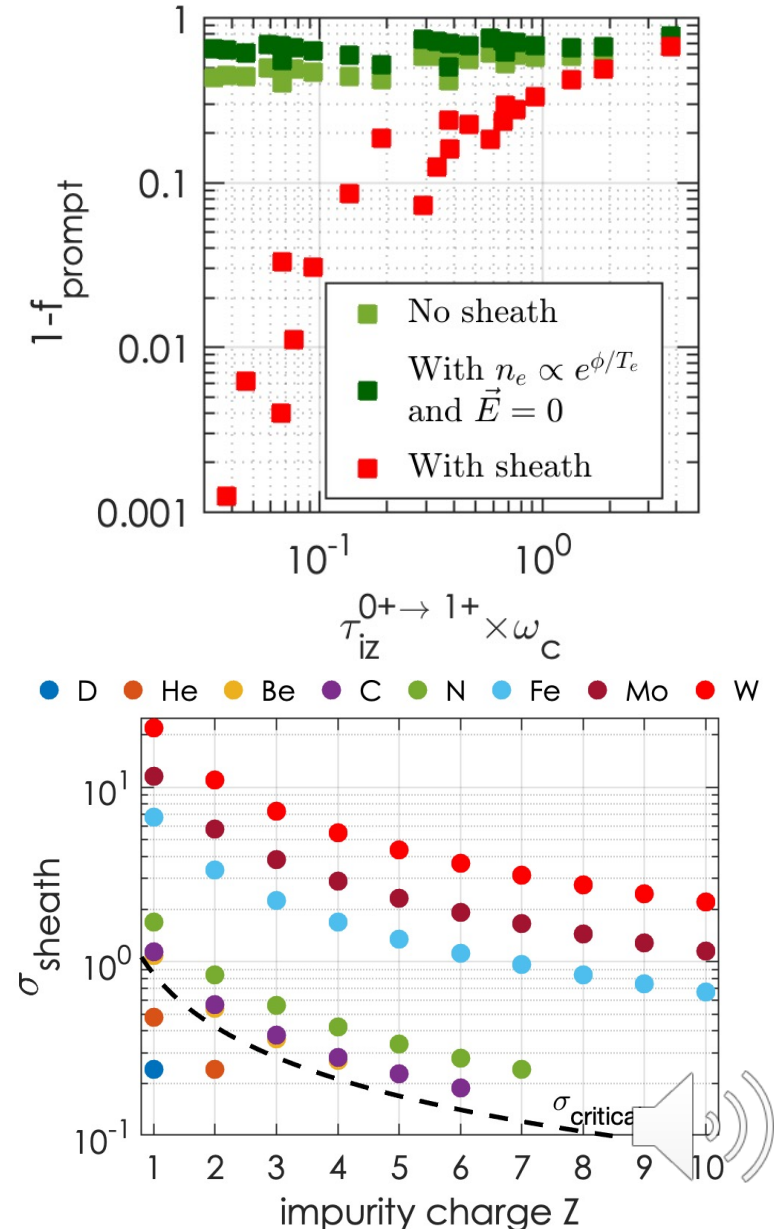
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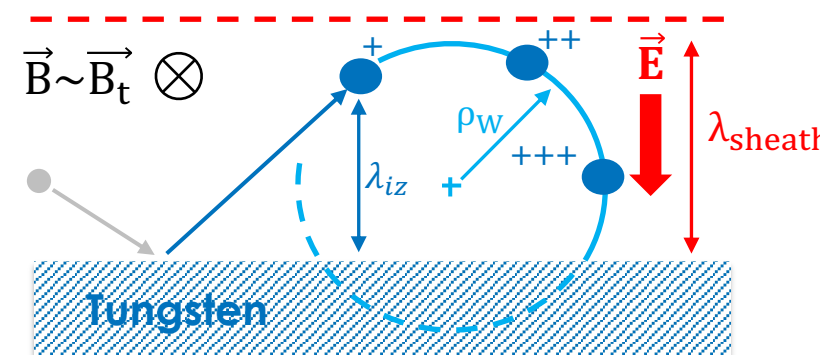
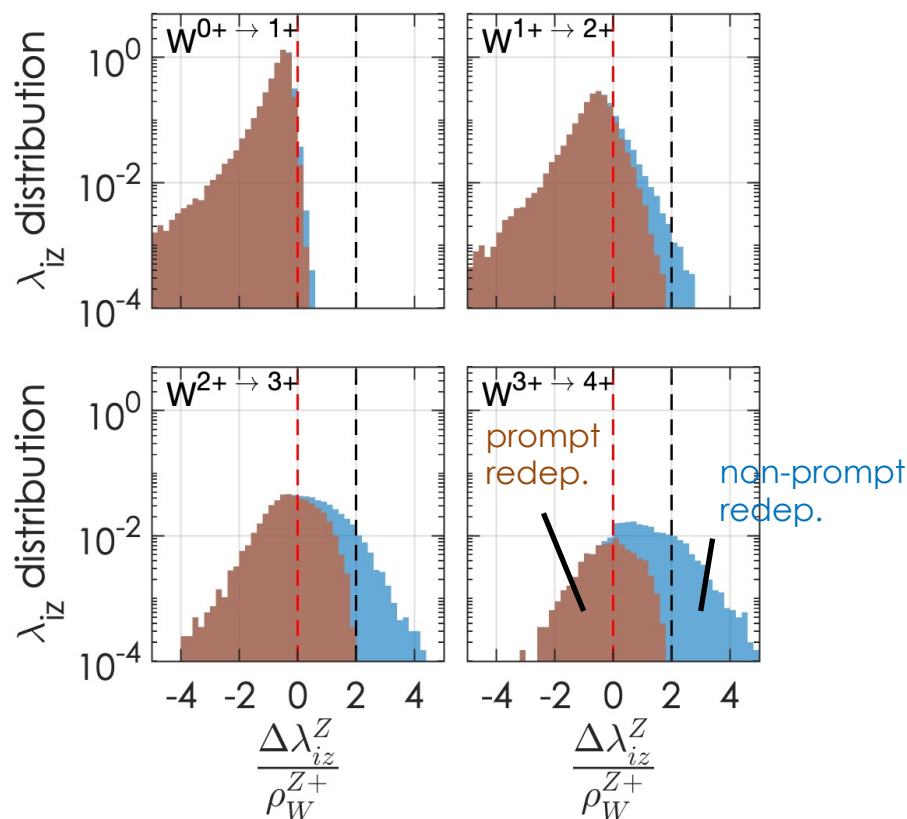
- Electric field remains much stronger than Lorentz force despite high W charge state due to large W mass

$$\sigma_{\text{sheath}}^W = \frac{Z \Lambda T_e}{\frac{1}{2} m_W \omega_c^2 \lambda_{\text{sheath}}^2} > \sigma_{\text{critical}} = \left(\frac{E_{\text{cutoff}}}{E_{\text{binding}}} \right) / \Lambda Z$$



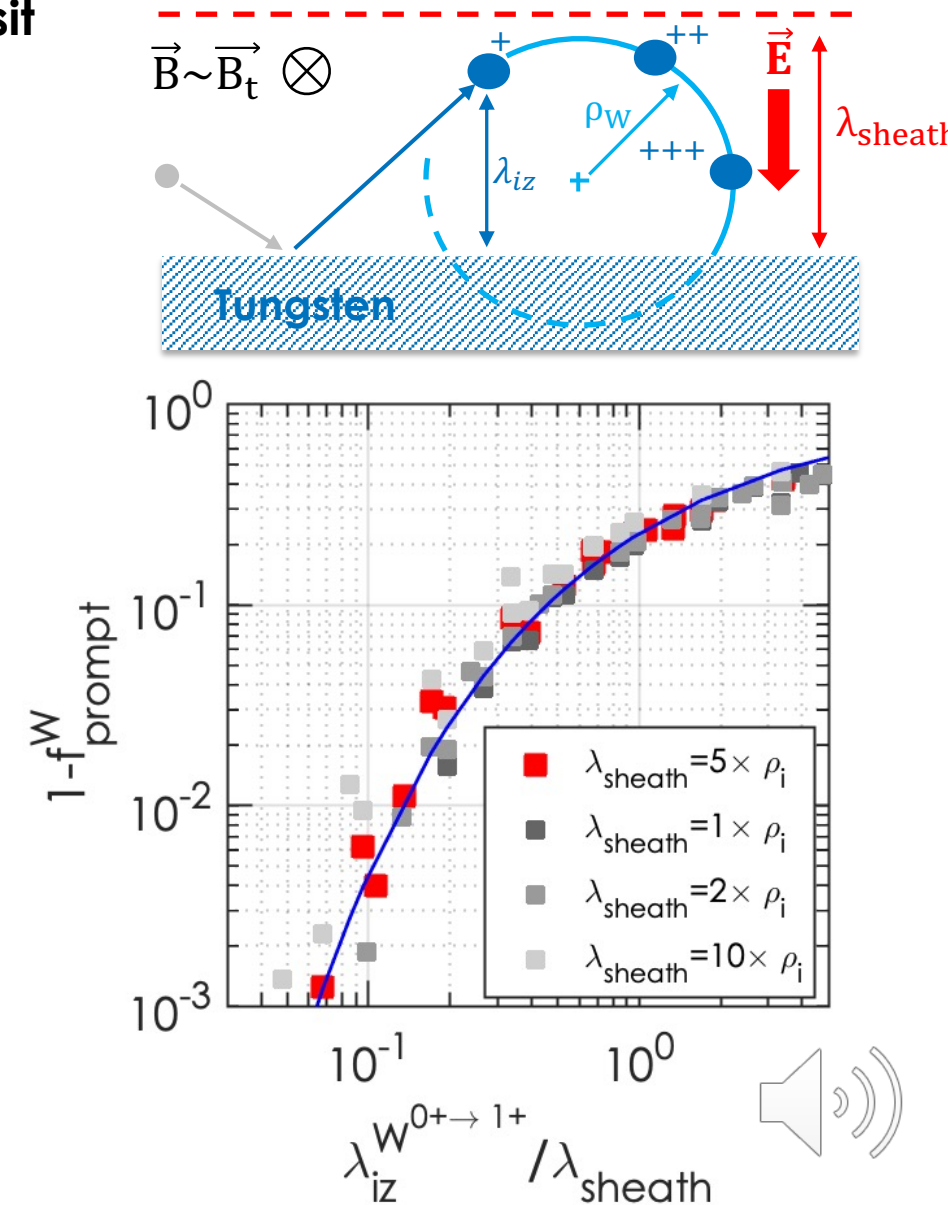
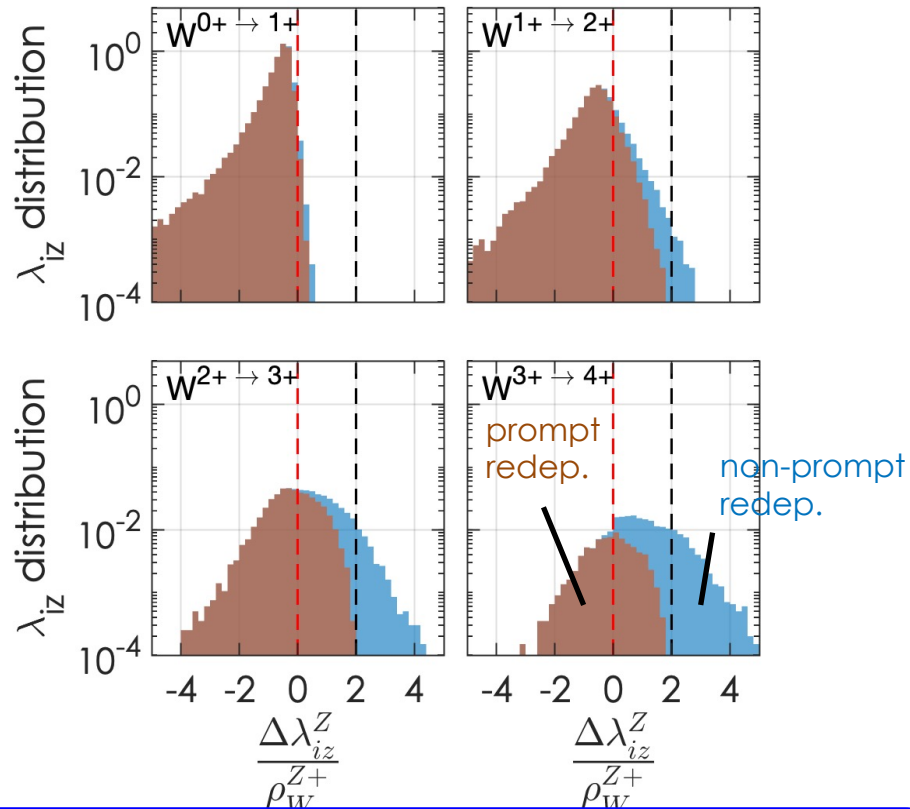
W prompt redeposition scales with the ratio of the neutral W ionization mean-free path over the sheath width

- Only W impurity ionizing out of the sheath do not promptly redeposit because of the strong sheath electric field



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- W prompt redeposition scales as the W neutral ionization mean-free path over the sheath width:

$$\frac{\lambda_{iz}^{0+ \rightarrow 1+}}{\lambda_{sheath}} = \frac{\langle v_W \rangle \tau_{iz}^{0+ \rightarrow 1+}}{\lambda_{sheath}}$$



New scaling law for W prompt redeposition with analytical formulation

- Consequently, the fraction $1 - f_{\text{prompt}}^W$ of W impurities non-promptly redeposited is correlated to the fraction f_{sheath}^W of W impurities ionized within the sheath:

$$1 - f_{\text{prompt}}^W \approx \frac{1}{2} \sqrt{1 - f_{\text{sheath}}^W}$$

- f_{sheath}^W can be analytically expressed¹:

$$f_{\text{sheath}}^W = \int_0^{\left(\frac{\lambda_{iz}^{W^{0+ \rightarrow 1+}}}{\lambda_{\text{sheath}}}\right)^{-1}} \Upsilon_{\xi_c}(\eta_b) d\eta_b \quad \text{where} \quad \xi_c = \frac{E_c}{E_b}$$



¹ [J. Guterl CPP 2020](#) ² [J. Guterl NME 2021](#)

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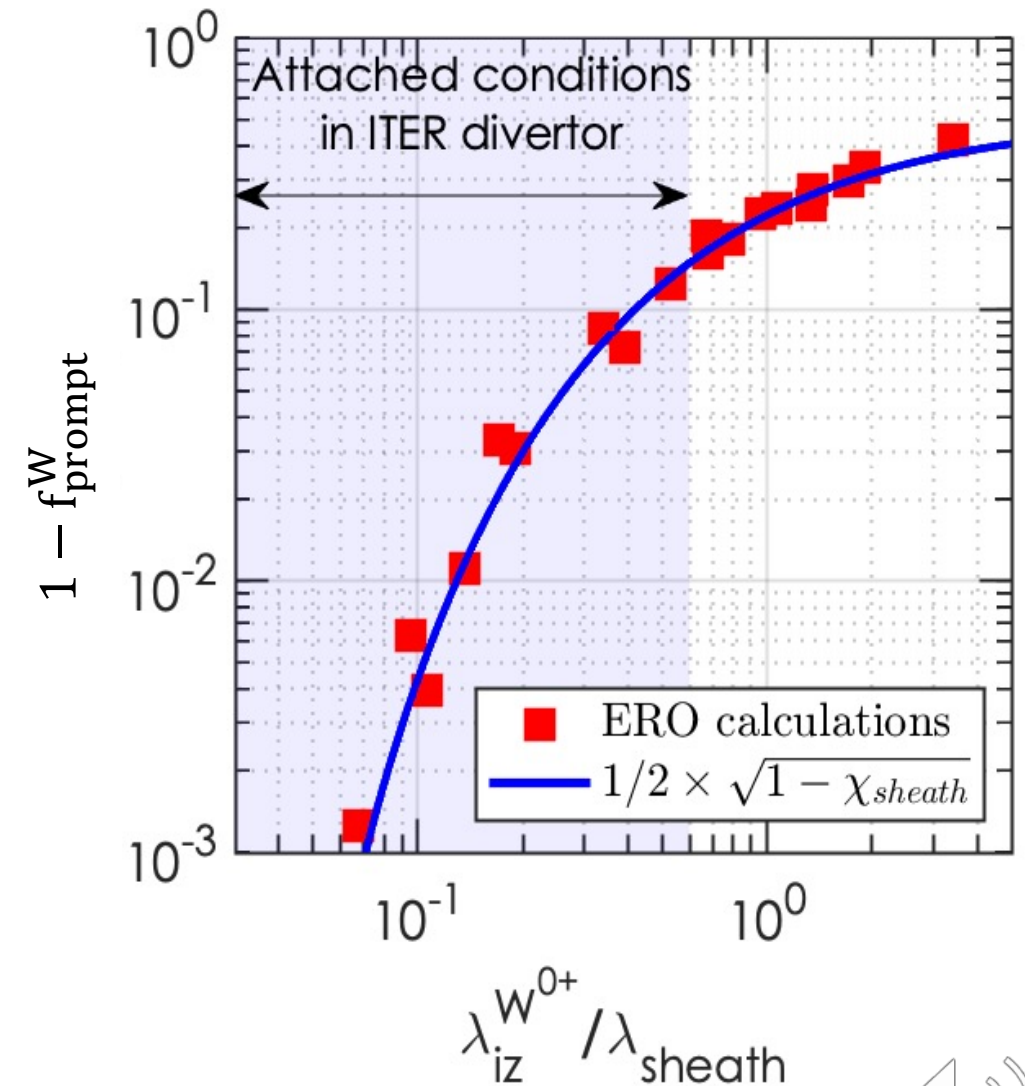
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- New robust analytical scaling law for W prompt redeposition²



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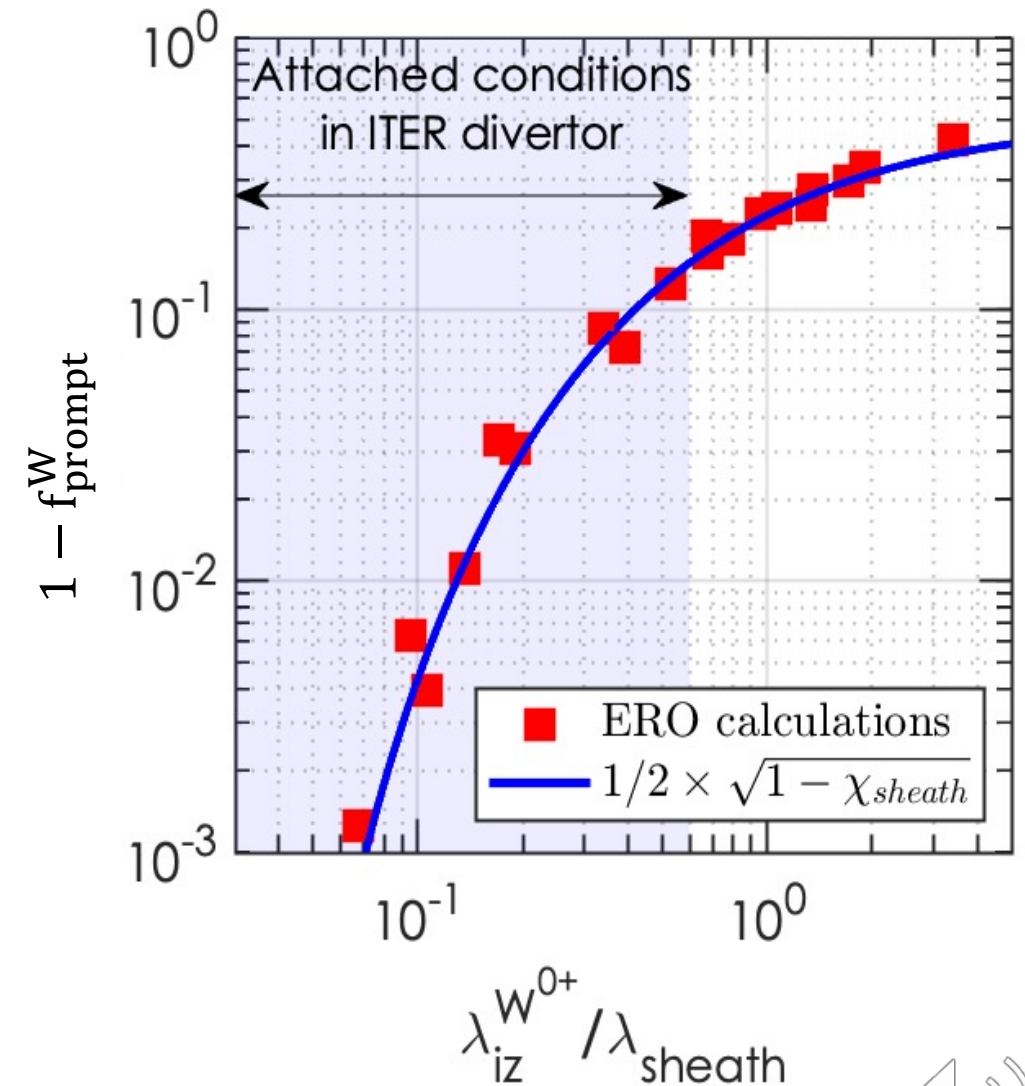
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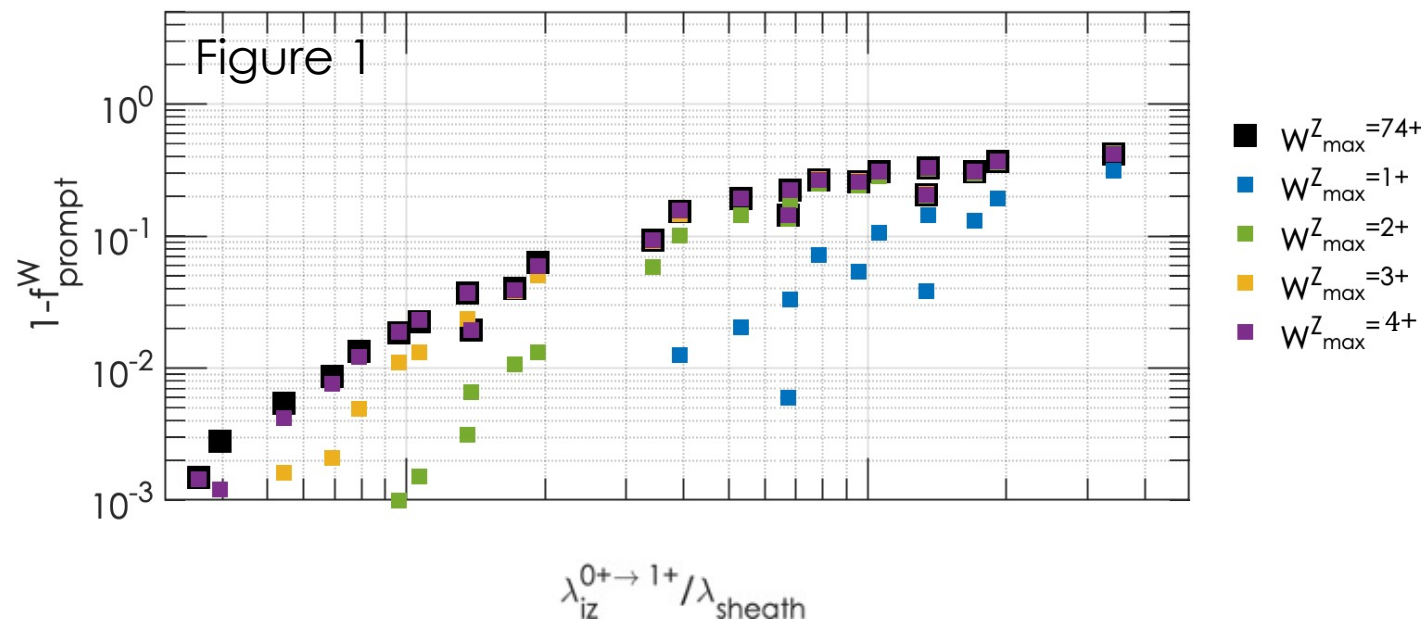
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- New robust analytical scaling law for W prompt redeposition²**
- Tungsten prompt redeposition governed by:**
 - $\lambda_{\text{iz}}^{W^{0+ \rightarrow 1+}}$: tungsten ionization rates
 - λ_{sheath} : width of the sheath
 - E_c : tail of the energy distribution of sputtered W particles, determined by energy of particles impinging on W PFCs



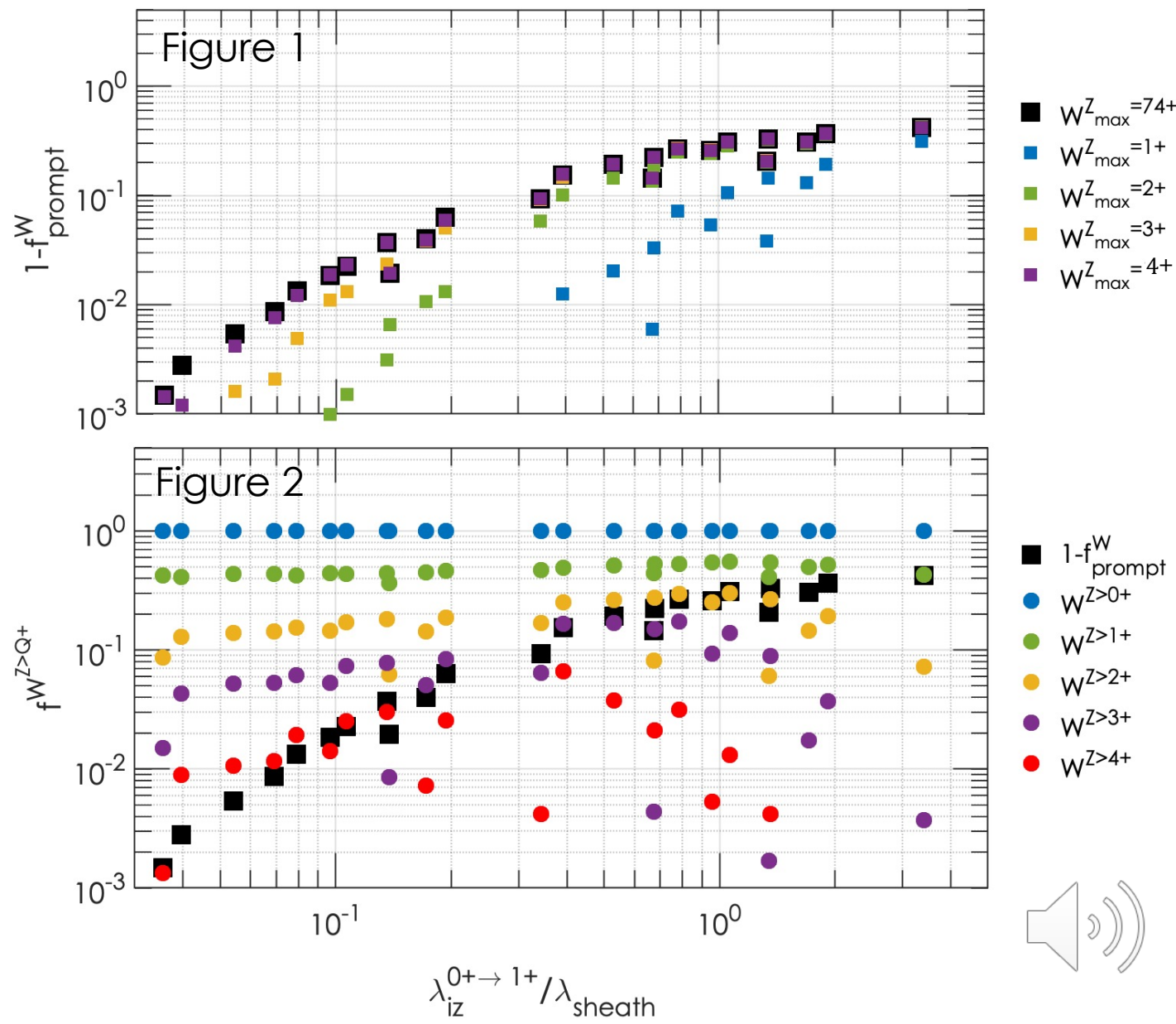
W prompt redeposition not directly correlated to the multiple ionizations of W impurities during their first gyro-orbit

- How to monitor tungsten prompt redeposition and net erosion in tokamak divertors?
- Multiple ionizations of W impurities during their first gyro-orbit still critical to allow W impurities to prevent re-entering in the sheath region and avoid prompt redeposition (fig. 1) ...



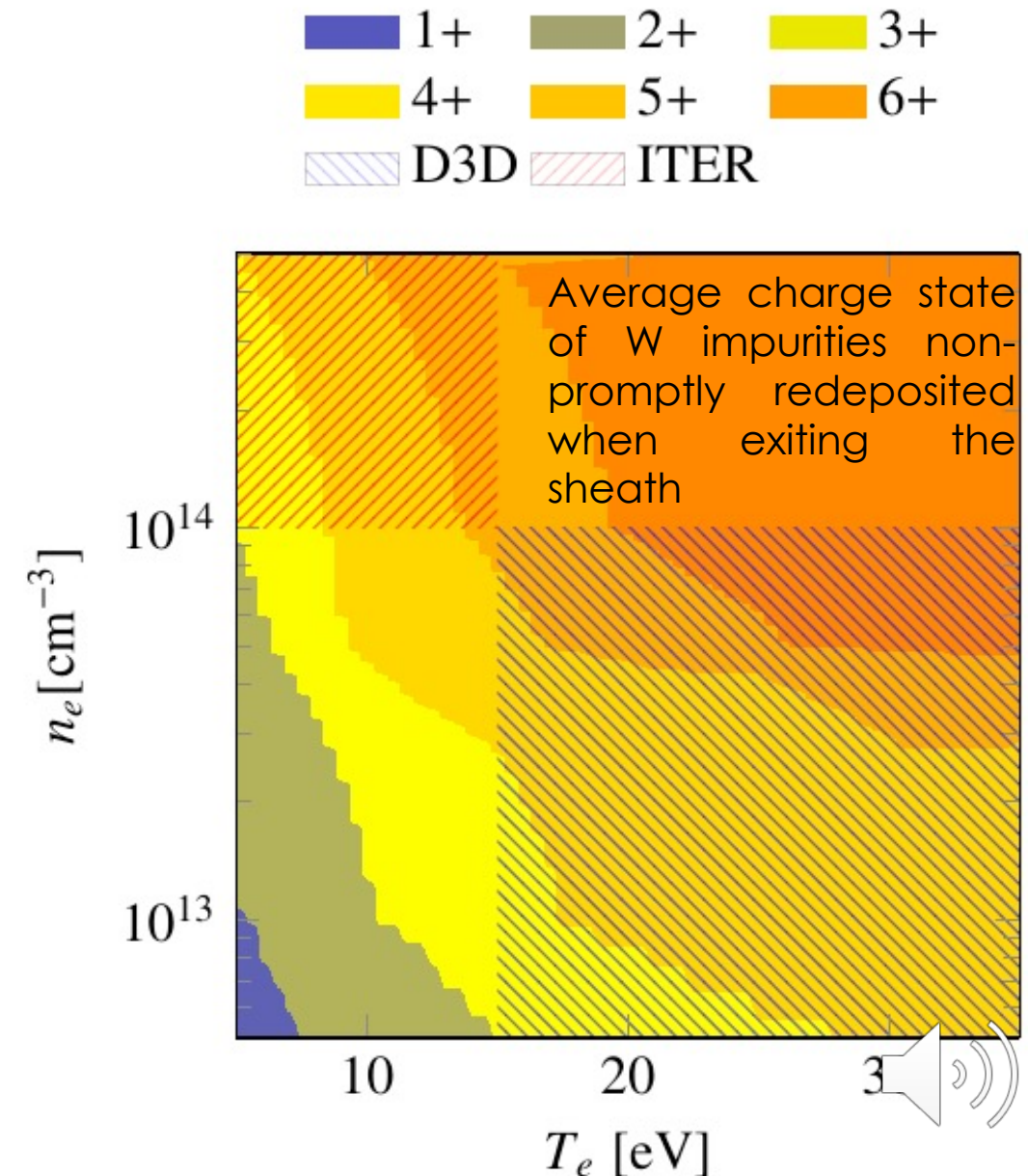
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- Multiple ionizations of W impurities during their first gyro-orbit still critical to allow W impurities to prevent re-entering in the sheath region and avoid prompt redeposition (fig. 1) ...
- ... but no global correlation between tungsten prompt redeposition and W ionization events across divertor conditions (fig. 2) !



In-situ monitoring of W net erosion in ITER divertor requires spectroscopic measurements of emissions from W-III, W-IV and W-V lines

- In absence of global correlation between ionizations and prompt redeposition of W impurities, spectroscopic measurement of multiple emission lines required to monitor W net erosion
- In-situ monitoring of W net erosion only possible through spectroscopic measurements of W-III, W-IV and W-V emission lines
- Dominant charge-states for non-promptly redeposited W impurities expected to be similar in current tokamaks, e.g. DIII-D, and in future fusion devices (ITER) operating at higher divertor plasma density



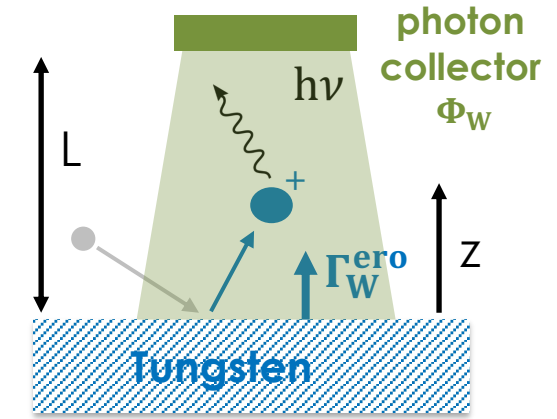
In-situ monitoring of W erosion through S/XB coefficients strongly affected by the ionization and emission of sputtered W impurities within the sheath region

- **W gross erosion flux given by** $\Gamma_W^{\text{ero}} = \int_0^L S_{\text{iz}}^{W^{0+} \rightarrow W^{1+}}(T_e, n_e) n_{W^{0+}} n_e dz$
- Introducing the photon emissivity coefficient $\sigma_{\text{photon}}^{W^{0+} \rightarrow 1+}$, W gross erosion flux can be inferred using the S/XB coefficient¹

$$\Gamma_W^{\text{ero}} = \int_0^L \underbrace{\frac{S_{\text{iz}}^{W^{0+} \rightarrow 1+}(T_e, n_e)}{\sigma_{\text{photon}}^{W^{0+} \rightarrow 1+}(T_e, n_e)}}_{\text{S/XB}^{W^{0+} \rightarrow 1+}} \times \underbrace{\sigma_{\text{photon}}^{W^{0+} \rightarrow 1+}(T_e, n_e) n_{W^{0+}} n_e dz}_{\Phi_W}$$

- It is usually assumed $\lambda_{\text{sheath}} < \lambda_{\text{iz}}^{W^{0+} \rightarrow 1+}$, such that $\Gamma_W^{\text{ero}} = \text{S/XB}^{W^{0+} \rightarrow 1+} \times \Phi_W$

- **But when** $\lambda_{\text{iz}}^{W^{0+} \rightarrow 1+} < \lambda_{\text{sheath}}$, $\Gamma_W^{\text{ero}} = \underbrace{\frac{\int_0^L S_{\text{iz}}^{W^{0+} \rightarrow 1+}(T_e, n_e(z)) n_{W^{0+}}(z) n_e(z) dz}{\int_0^L \sigma_{\text{photon}}^{W^{0+} \rightarrow 1+}(T_e, n_e(z)) n_{W^{0+}}(z) n_e(z) dz}}_{< \text{S/XB}^{W^{0+} \rightarrow 1+} >_{\text{eff}}} \times \Phi_W$



¹ K. Behringer PPCF 1989

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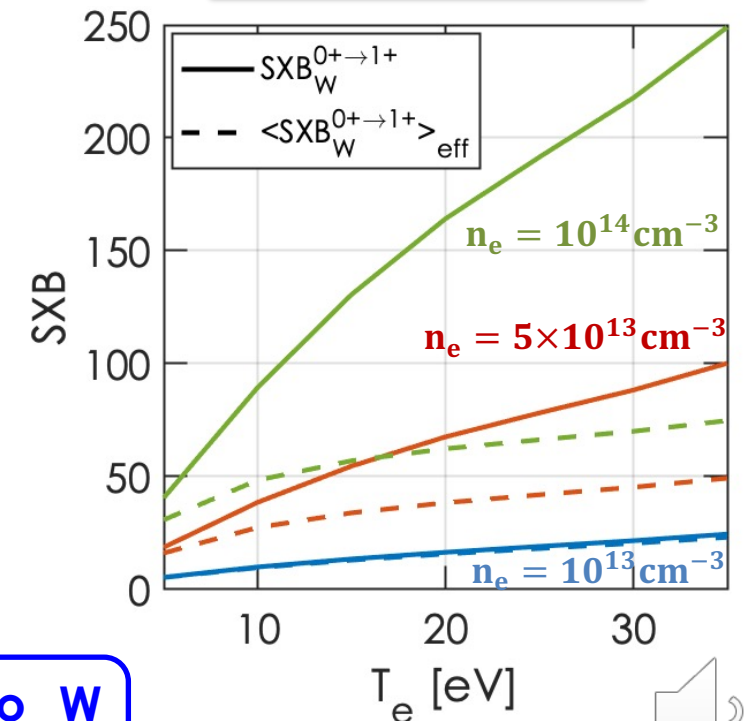
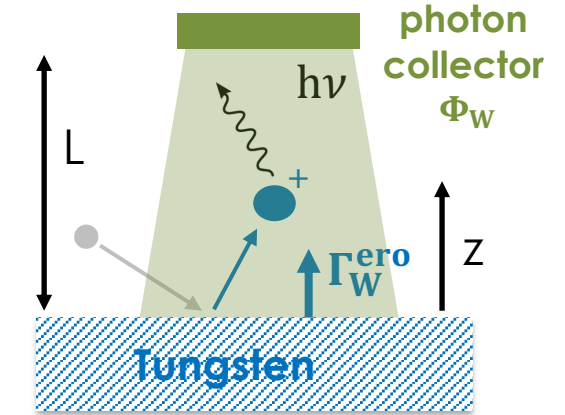
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Reduction of the effective SXB coefficients at high n_e due to W ionization and emission within the electric sheath²



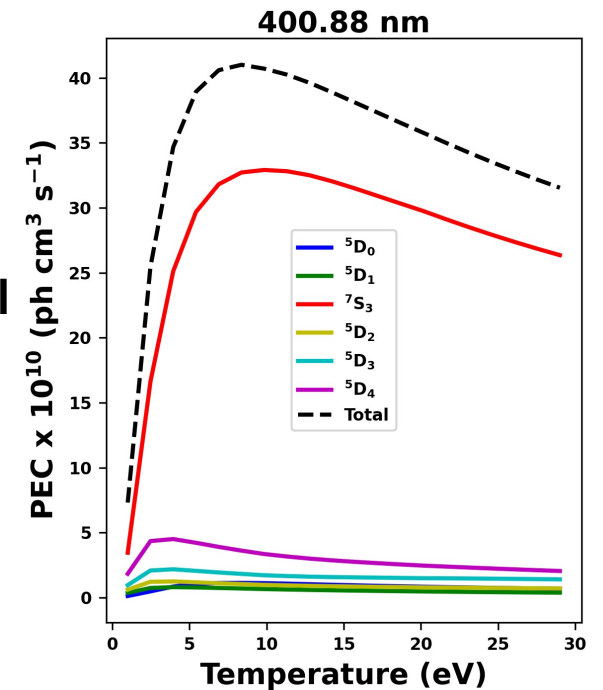
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Transient W metastable states also shown to impact S/XB coefficients

- Using excitation rates from a new non-perturbative Dirac R-matrix electron-impact calculation for W^I¹, it can be shown³:
 - W I PECs for intense spectra lines are dominated by a single metastable level^{*,**}**
 - Total value of the ionization coefficient for neutral tungsten is relative insensitive to changes in the metastable fraction**

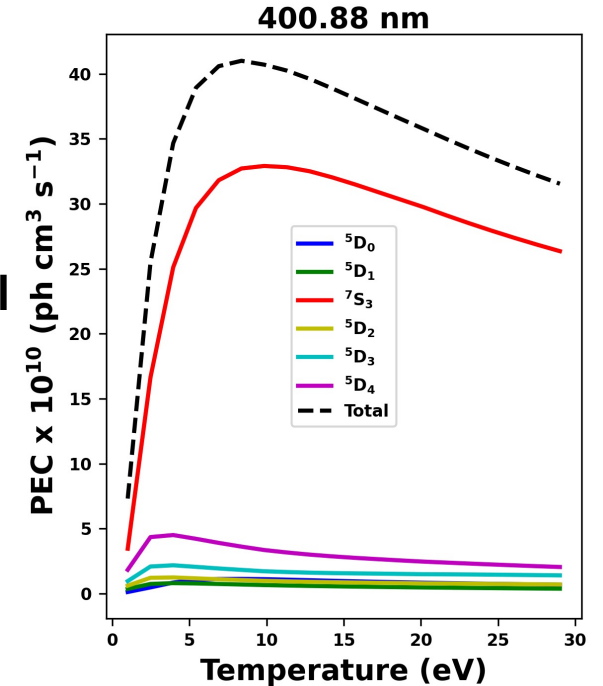
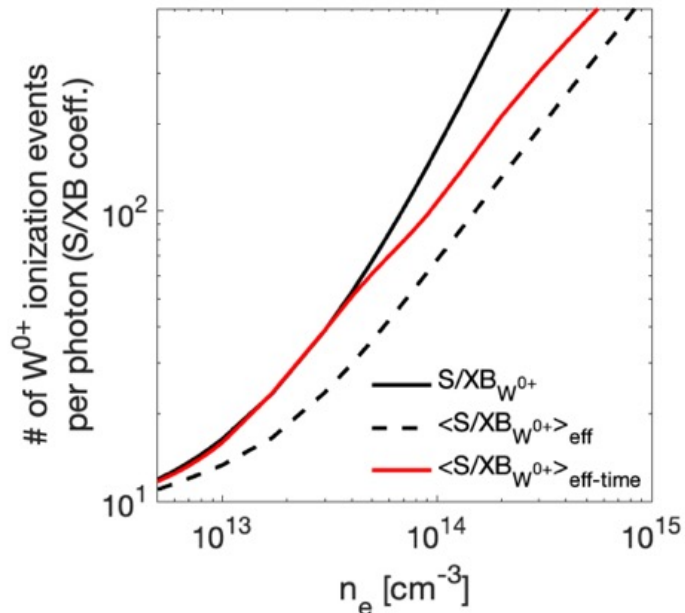


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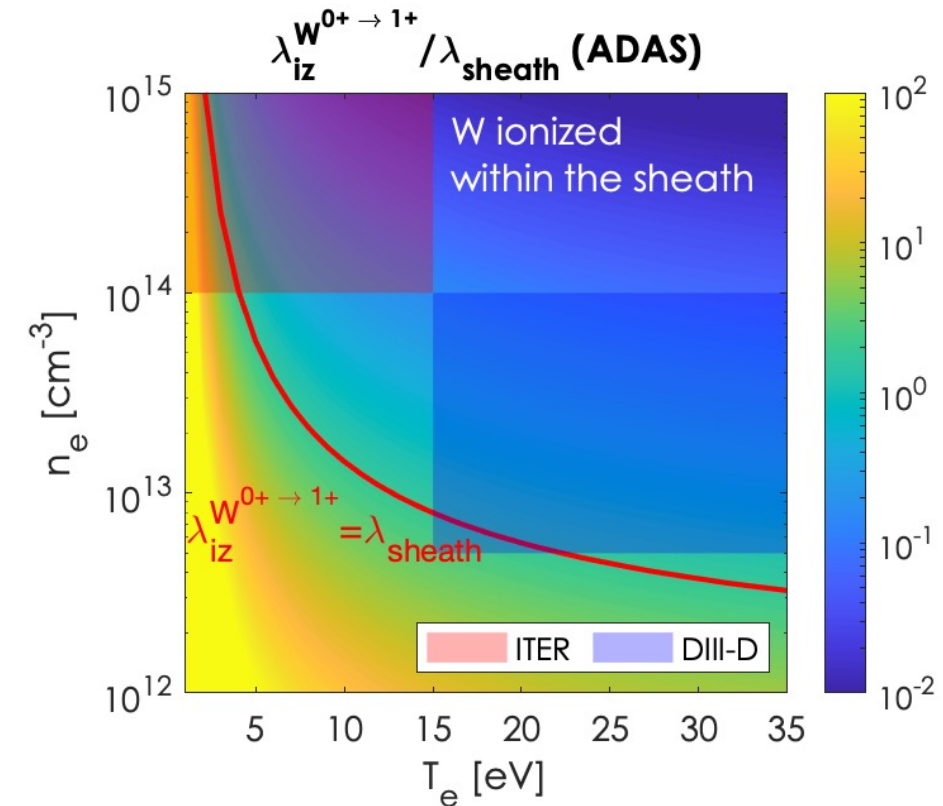
- Using the new collision-radiative solver ColRadpy³:
 - Time dependent effective S/XBs can deviate significantly from steady-state due to non-equilibrium population of metastable state**

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Predictive modeling for W net erosion in divertors can be validated with innovative experiments in DIII-D

- W prompt redeposition governed by the ratio of W neutral ionization mean-free path over the sheath width: $\lambda_{iz}^{W^{0+ \rightarrow 1+}} / \lambda_{\text{sheath}}$
- Divertor plasma conditions in DIII-D significantly different than in ITER but values of $\lambda_{iz}^{W^{0+ \rightarrow 1+}} / \lambda_{\text{sheath}}$ are similar**
 - Regime of W prompt redeposition similar for DIII-D and ITER divertor conditions**

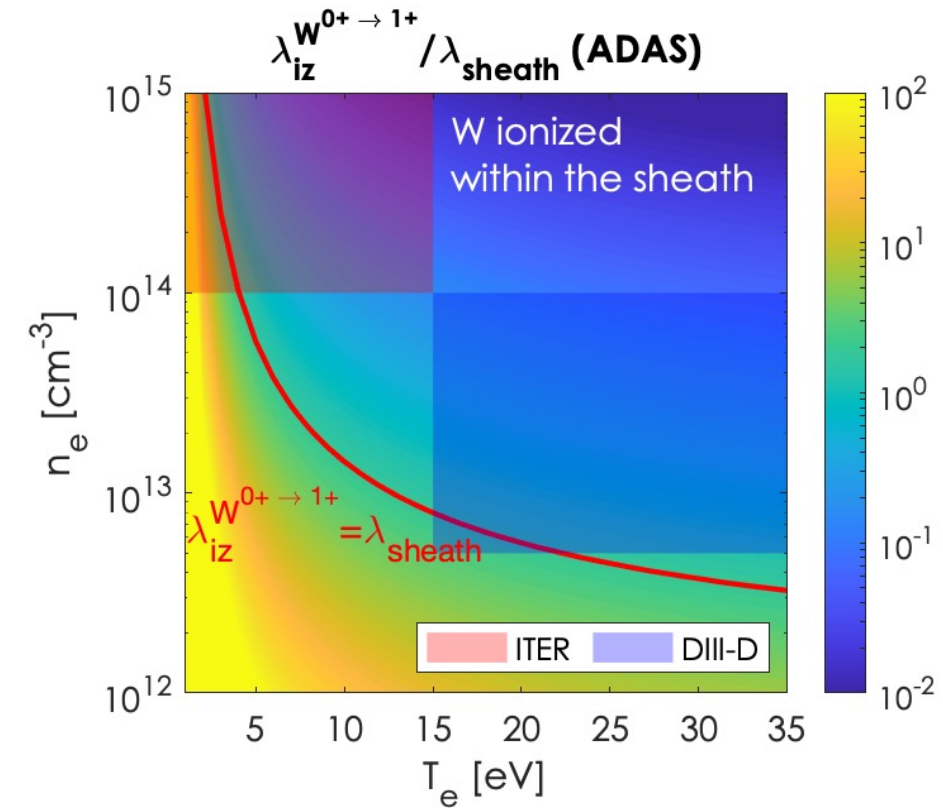


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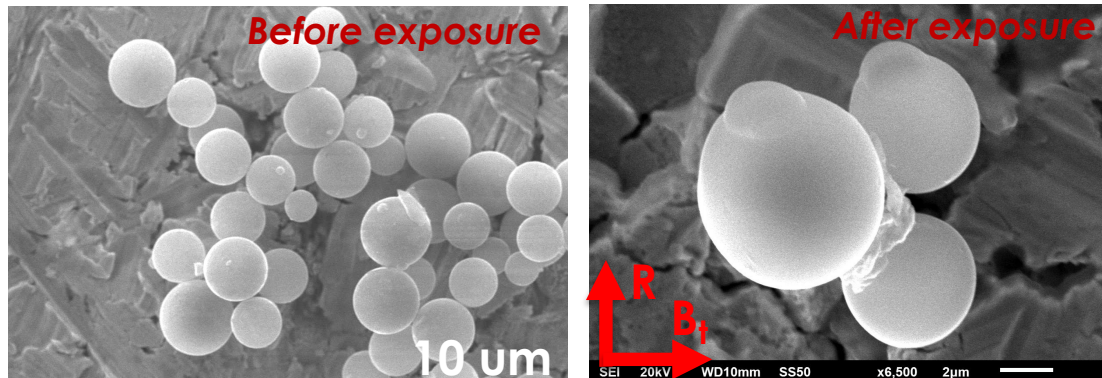
Experiments conducted at DIII-D to validate predictive modeling of W net erosion and assess physics parameters governing W net erosion:

- Experimental estimations of the sheath width
- Direct measurement of W net erosion



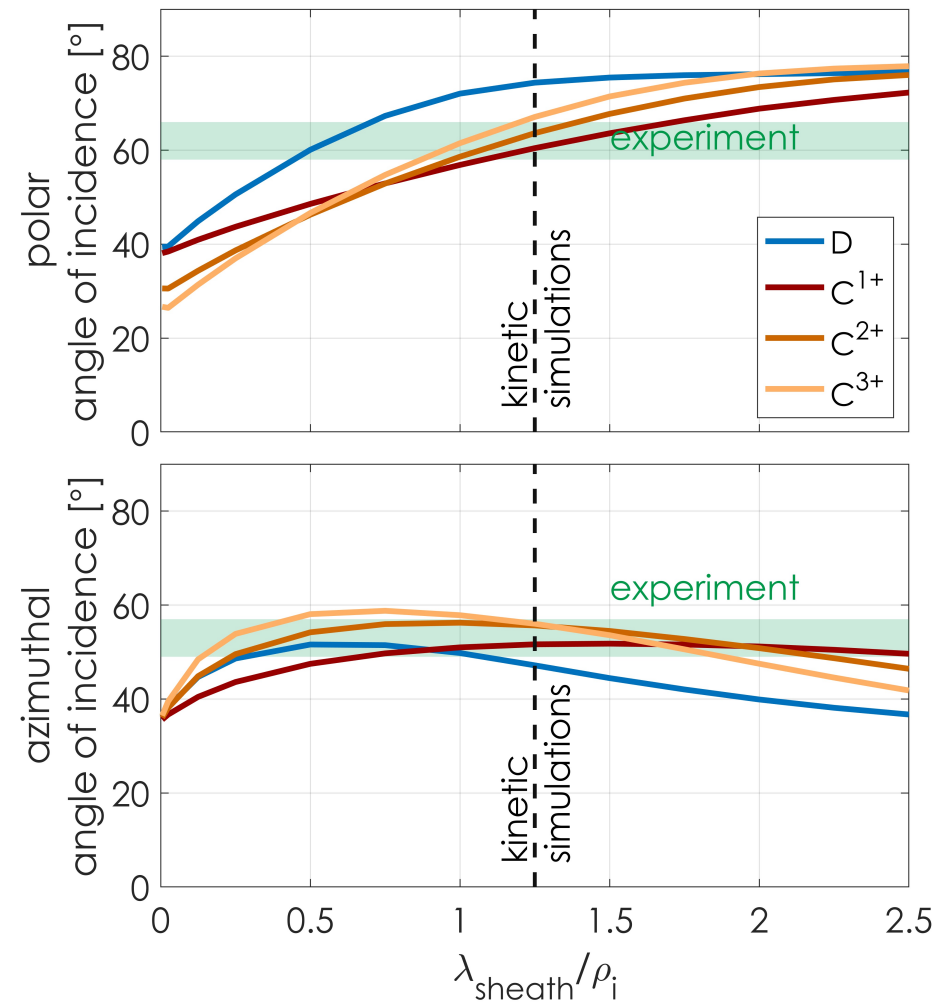
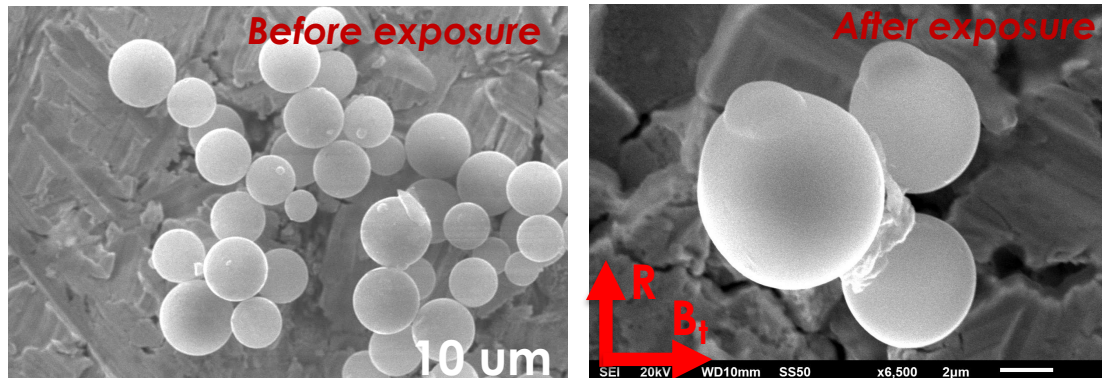
Experimental estimation of the sheath width in the DIII-D divertor from the erosion of carbon micro-spheres

- Sheath width λ_{sheath} at divertor targets numerically estimated with PIC and kinetic simulations¹: $\lambda_{\text{sheath}} = 1.2 \times \rho_i$
- **Innovative experiments conducted in DIII-D to assess the sheath width :**
 - Angles of incidence of C impurities on divertor target inferred from small C deposition caps observed on C micro-spheres after exposure
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- λ_{sheath} inferred from the measurement of the angle of incidence of C ions impinging on divertor target in excellent agreement with kinetic simulation¹ of the sheath on divertor targets!
- Similar experiments conducted with micro-trenches



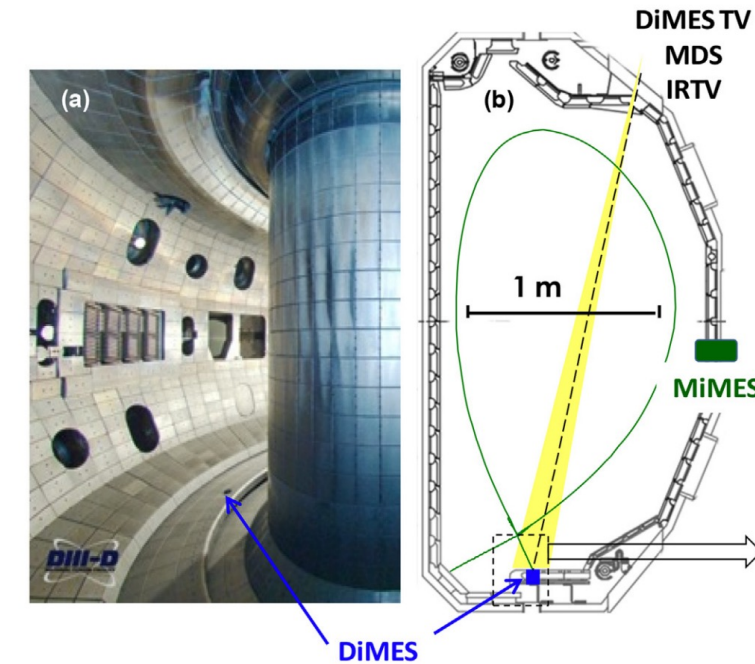
Experimental validation of model for W prompt redeposition and net erosion in DIII-D divertor with DiMES

- The Divertor Material Evaluation System (DiMES) allows for exposure of material samples in the lower divertor of DIII-D under well-diagnosed plasma conditions
- Experimental estimations of W net erosion through small/large dots DiMES experiments in DIII-D^{1,2}:

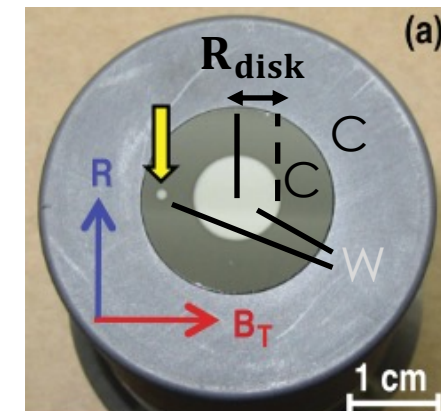
$$\langle \Gamma_W^{\text{net}} \rangle_{R_{\text{disk}}} = \Gamma_W^{\text{gross}} (1 - \xi_{\text{redep}}(R_{\text{disk}}))$$

ξ_{redep} : Fraction of W redeposited on W sample

- Strong dependency of ξ_{redep} on R_{disk} when $R_{\text{disk}} \sim \lambda_{\text{redep}} \sim 1\text{mm}$



DiMES with small and large W dots¹



¹ [D.Rudakov Physica Scripta 2014](#) ² [R. Ding Nuclear Fusion 2016](#)



Experimental validation of model for W prompt redeposition and net erosion in DIII-D divertor with DiMES

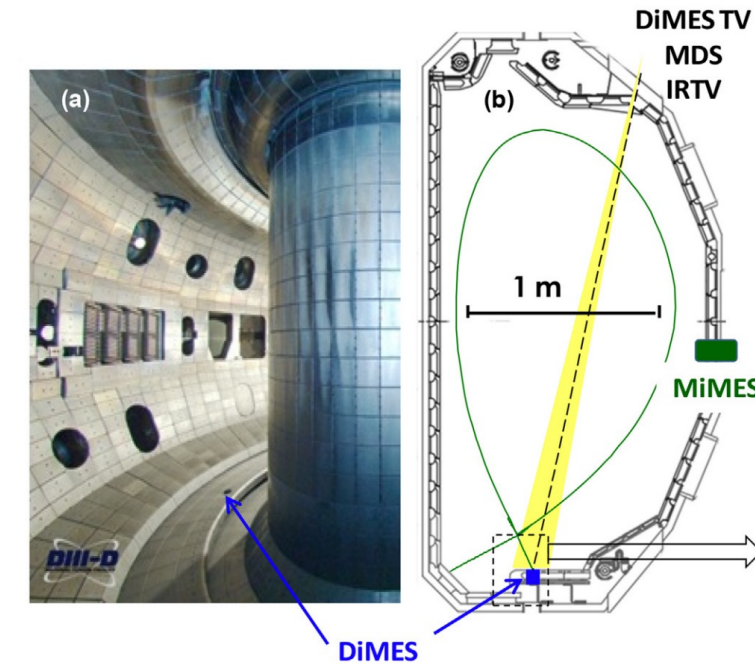
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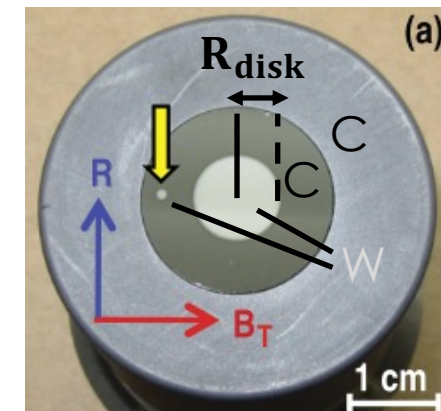
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- Experimental validation of model for W prompt redeposition and net erosion through comparison of net erosion from small and large W dots exposed in DIII-D divertor with DiMES**

$$\frac{\langle \Gamma_W^{\text{net}} \rangle_{\text{large disk}}}{\langle \Gamma_W^{\text{net}} \rangle_{\text{small disk}}} = \frac{\Gamma_W^{\text{gross}} (1 - \xi_{\text{redep}}(R_{\text{large}}))}{\Gamma_W^{\text{gross}} (1 - \xi_{\text{redep}}(R_{\text{small}}))}$$



DiMES with small and large W dots¹



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Reduced model of W prompt redeposition and net erosion in agreement with experimental measurements of W net erosion in DIII-D

- Reduced model with few characteristic parameters ($\beta_{\text{redep}}, \lambda_{\text{redep}}$) developed to analyze small/large dots DiMES experiments

W ionization rates, energy distribution, n_e , T_e , sheath, B field

$\beta_{\text{redep}}, \lambda_{\text{redep}}$

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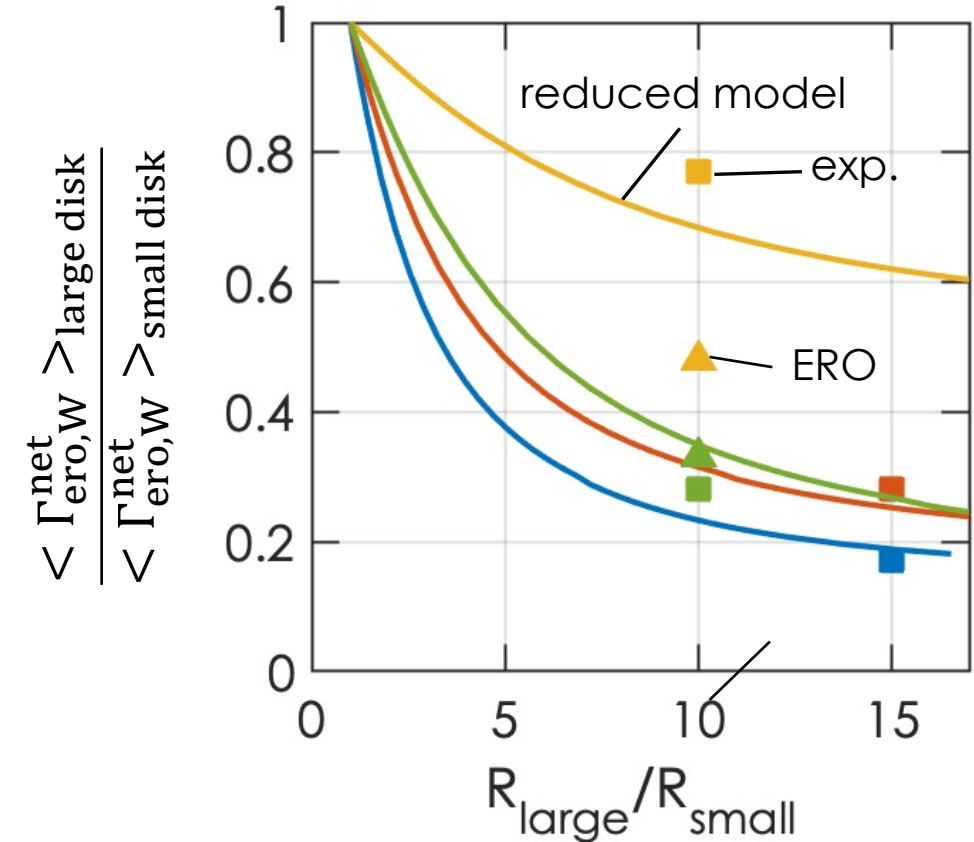
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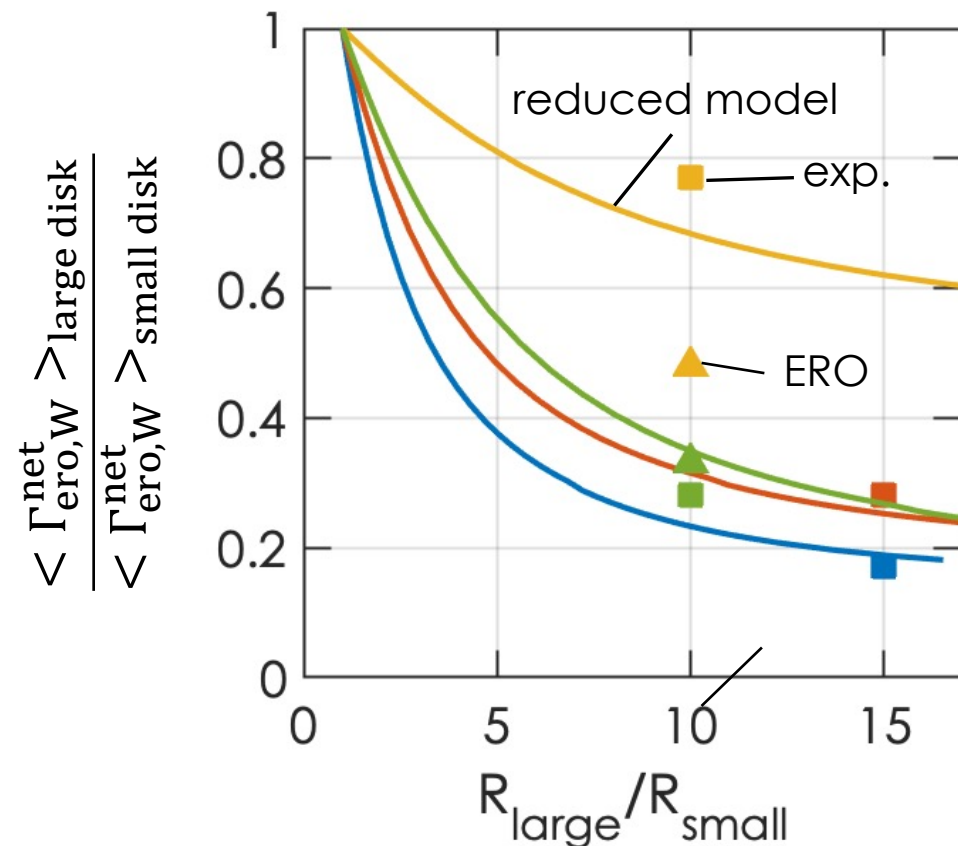
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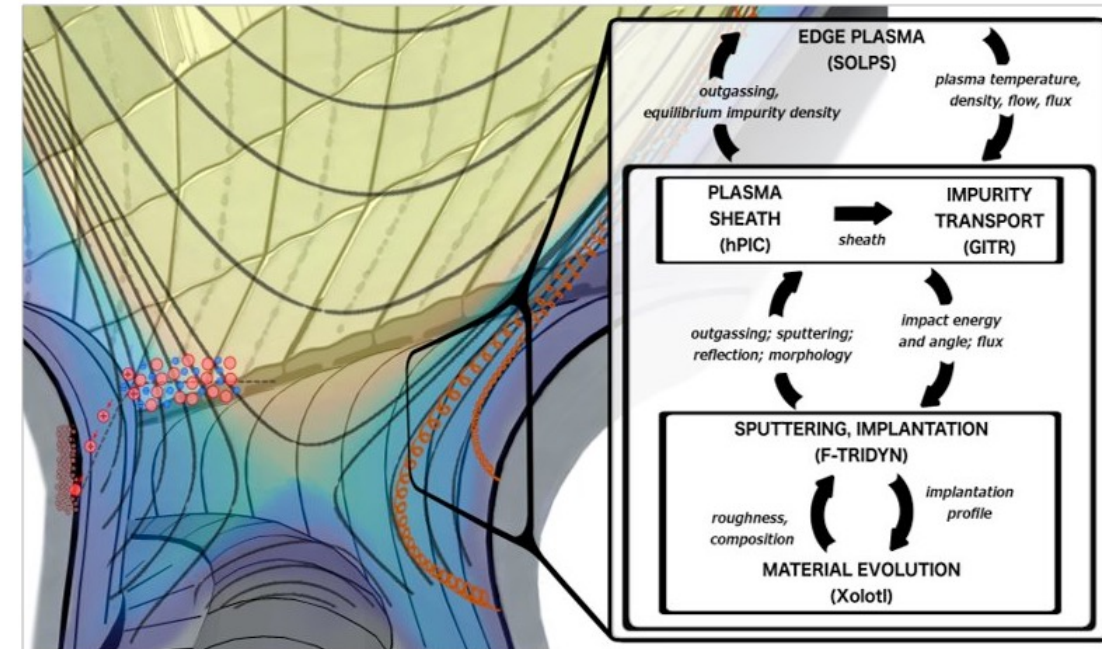
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- Reduced model in good agreement with experimental measurements in various plasma conditions and with comprehensive ERO model¹
- Experimental framework available in DIII-D for quantitative assessment of critical parameters controlling W prompt redeposition and net erosion (W ionization rates, PEC)



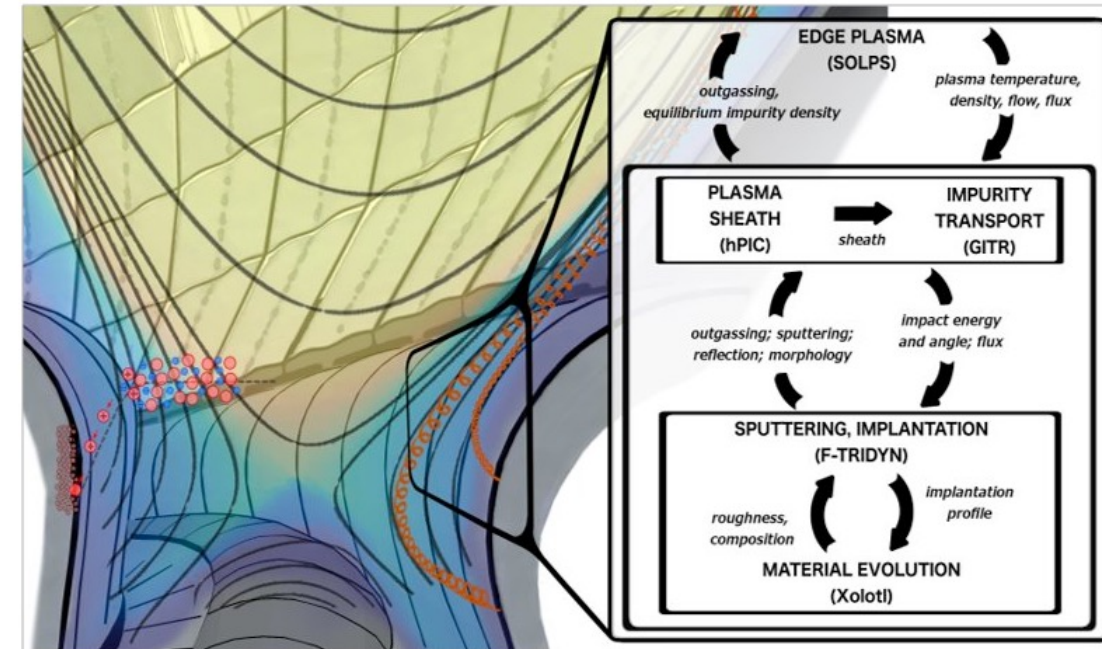
PSI-SciDAC: development and integration of HPC simulation tools capable of predicting W PFCs lifetime and impact on plasma performance

- Large US multi-institutions project led by B. Wirth to predict the performance and impact of dynamic W PFC material¹
 - Development and integration of suite of coupled plasma and materials modeling tools to predict **multi-scale** PFC evolution and feedback to the boundary plasma
 - Active framework to apply **uncertainties quantification to PMI codes and workflow**²
- Recent focus on modeling of **Be-W system**
- An example of workflow for W divertor PFCs^{3,4}:
 - Boundary plasma: SOLPS
 - Plasma sheath: hPIC
 - 3D trace impurity transport: GITR
 - Erosion, surface morphology: F-TryDyn
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- HPC simulations tools developed in the PSI-SciDAC project can support ITER goals for PMI modeling
 - Framework for collaboration?



Summary

- Tungsten(W) prompt redeposition in tokamak divertors is mainly governed by the ratio of the ionization mean-free path of sputtered neutral W particles over the sheath width:
 - *New scaling law for W prompt redeposition with analytical formulation*
- In-situ monitoring of W net erosion in divertors requires *monitoring photon emissions associated with the ionization of W impurities in charge states $Z > 2+$, typically W-III, W-IV and W-V lines for ITER*
- Parameter governing W prompt redeposition has similar values for divertor plasma conditions in DIII-D experiments and in ITER far-SOL:
 - *Experiments conducted at DIII-D to validate predictive modeling of W net erosion in ITER divertor and beyond*
 - Measurement of sheath width from erosion of carbon micro-spheres
 - Measurement of W prompt redeposition through small/large dots experiments
- HPC simulation tools for W/Be PMI modeling W available through the PSI2-SciDAC project

