

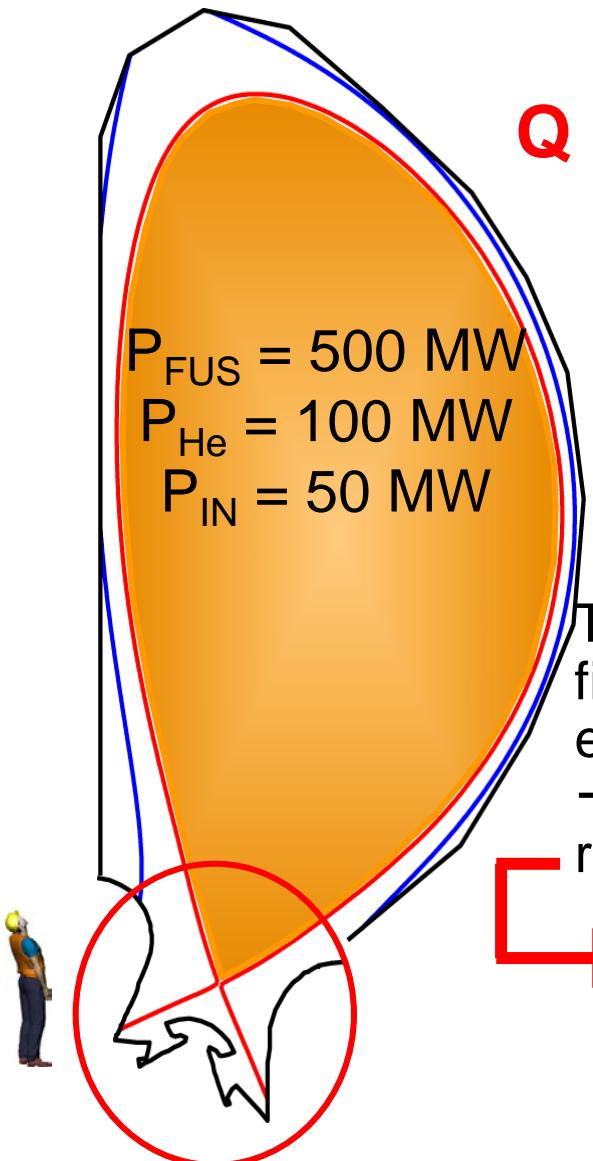
Atomic, Molecular And PMI Database In Current Edge Plasma Transport Codes, And Forward Sensitivity Analysis

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ITER: Balance of power



$$Q = \frac{\text{Fusion power}}{\text{Input power}} = \frac{500 \text{ MW}}{50 \text{ MW}} = 10$$



100 MW

Trapped by the magnetic field (He^{2+}) → gives its energy up to the plasma → maintains the fusion reactions

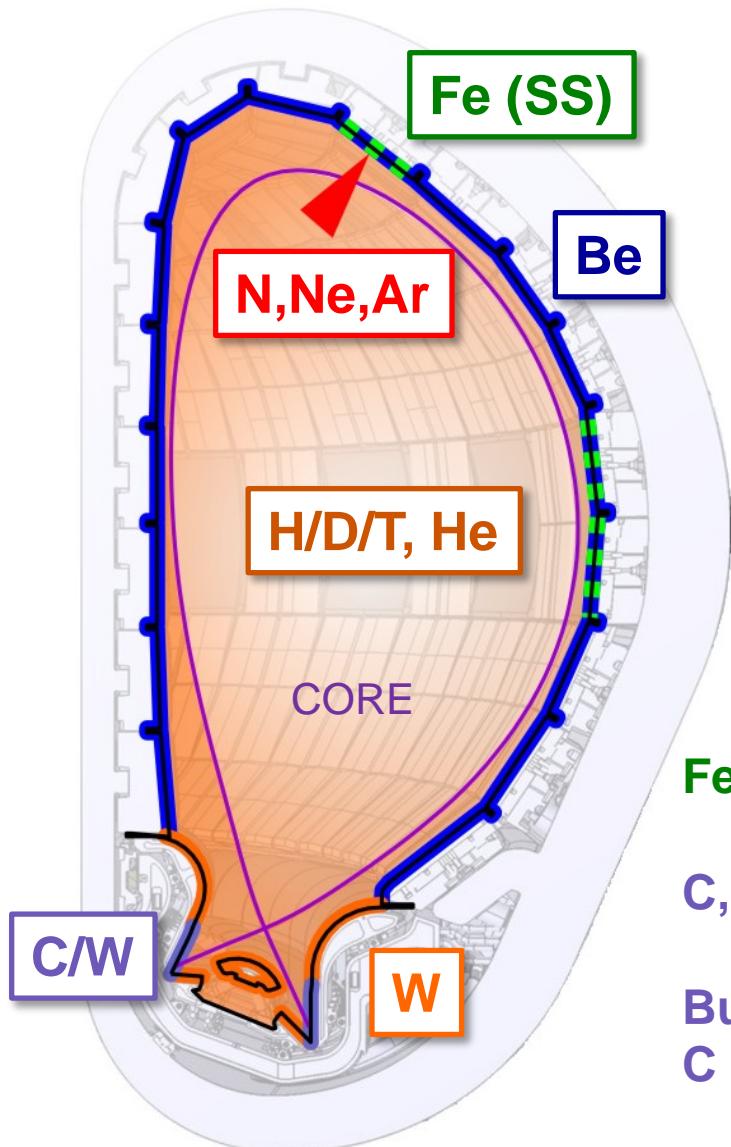
400 MW

Escapes from the plasma (no electric charge) → absorbed in the blanket surrounding the plasma

Must be exhausted from the plasma

The “divertor”

List of the main elements relevant to the ITER plasma



Fe/SS, 2011: armour at
diagnostics port plugs
C, 2013: replaced by
all W divertor
But:
C in: W7X (2016),
JT-60SA (2019),...

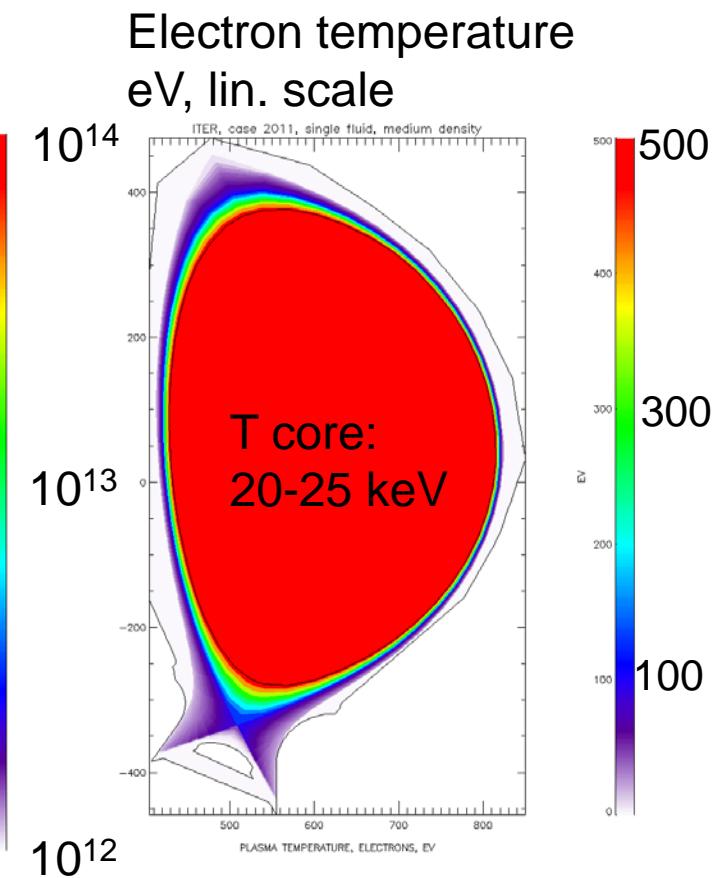
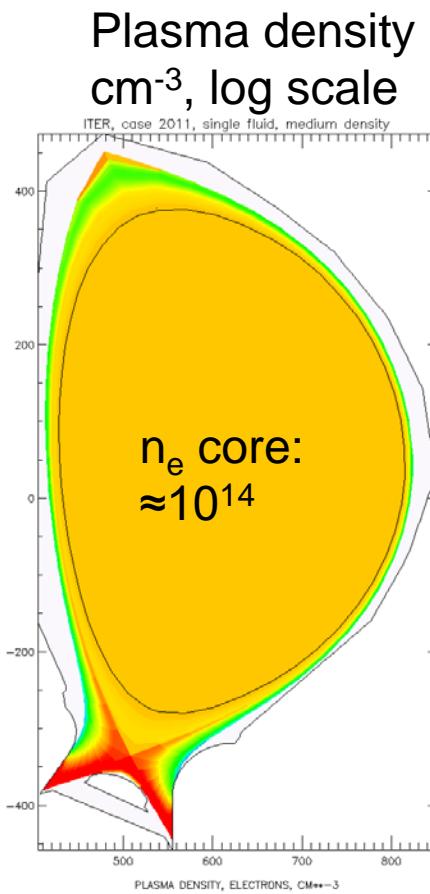
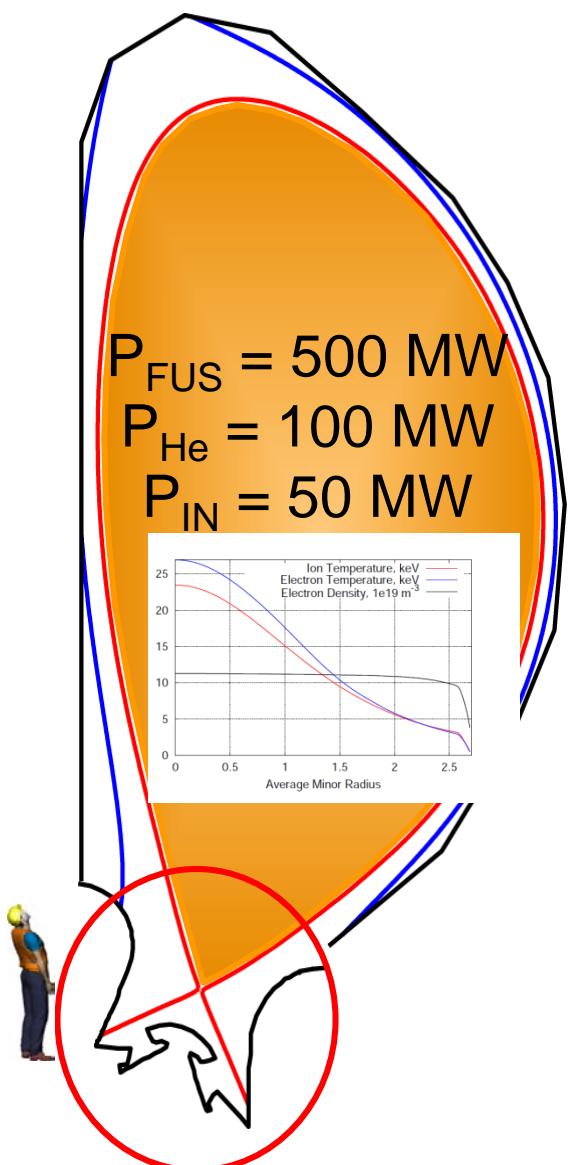
Outline

- A: Core plasma: (beam-) spectroscopy
strongly stripped ions, often: H, He-like
- B: Plasma boundary: spectroscopy plus: powerful plasma flows,
particle exhaust, heat exhaust, machine availability
- C: Typical parameters (electron density, temperature)
- D: A&M data sensitivity (\rightarrow uncertainty propagation)
 - 1: status: A&M data: sensitivity analysis in 0D plasma chemistry models.
 - 2: outlook: A&M data: sensitivity analysis in current edge plasma flow models ???

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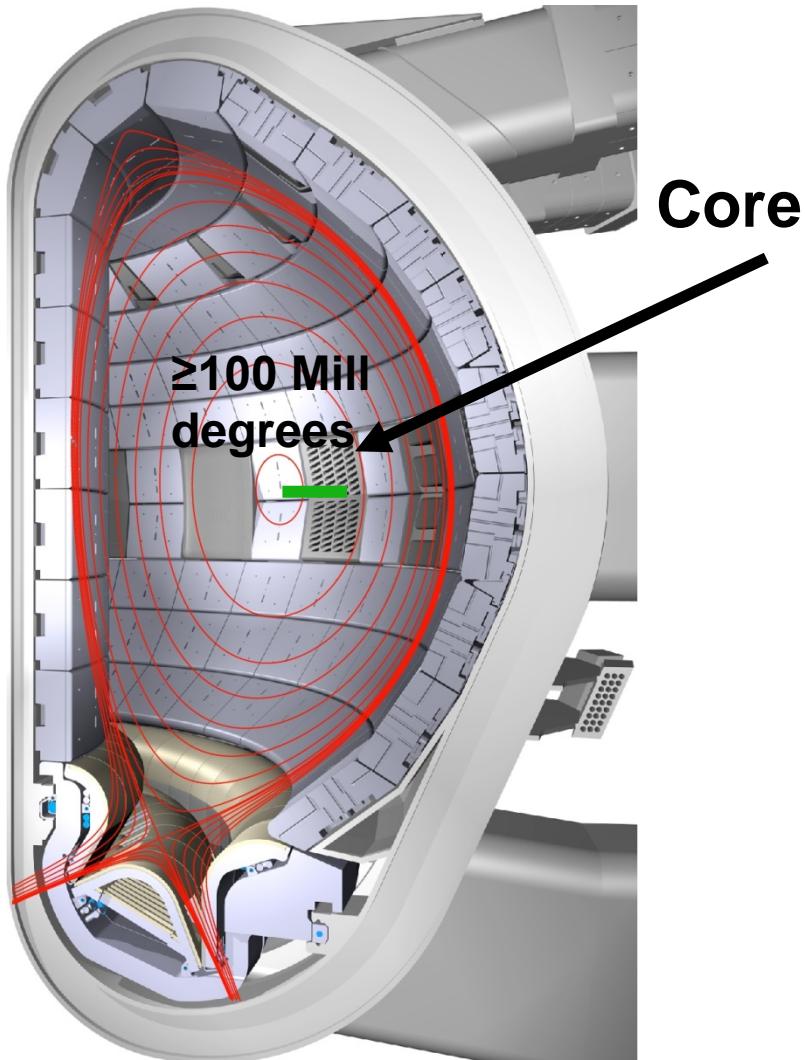
ITER, 500 MW fusion power, n_e , T_e



Relative importance of plasma flow forces over chemistry and PWI

$$\frac{\partial}{\partial t} n_i + \vec{\nabla} \cdot (n_i \vec{V}_i) = S_{n_i}$$

div($n v_{||}$) + div($n v_{\perp}$) = ionization/recombination/charge exchange



- All chemical bonds broken,
- (turbulent) cross field flow, D_{\perp} , V_{\perp}

(advanced plasma scenario development)

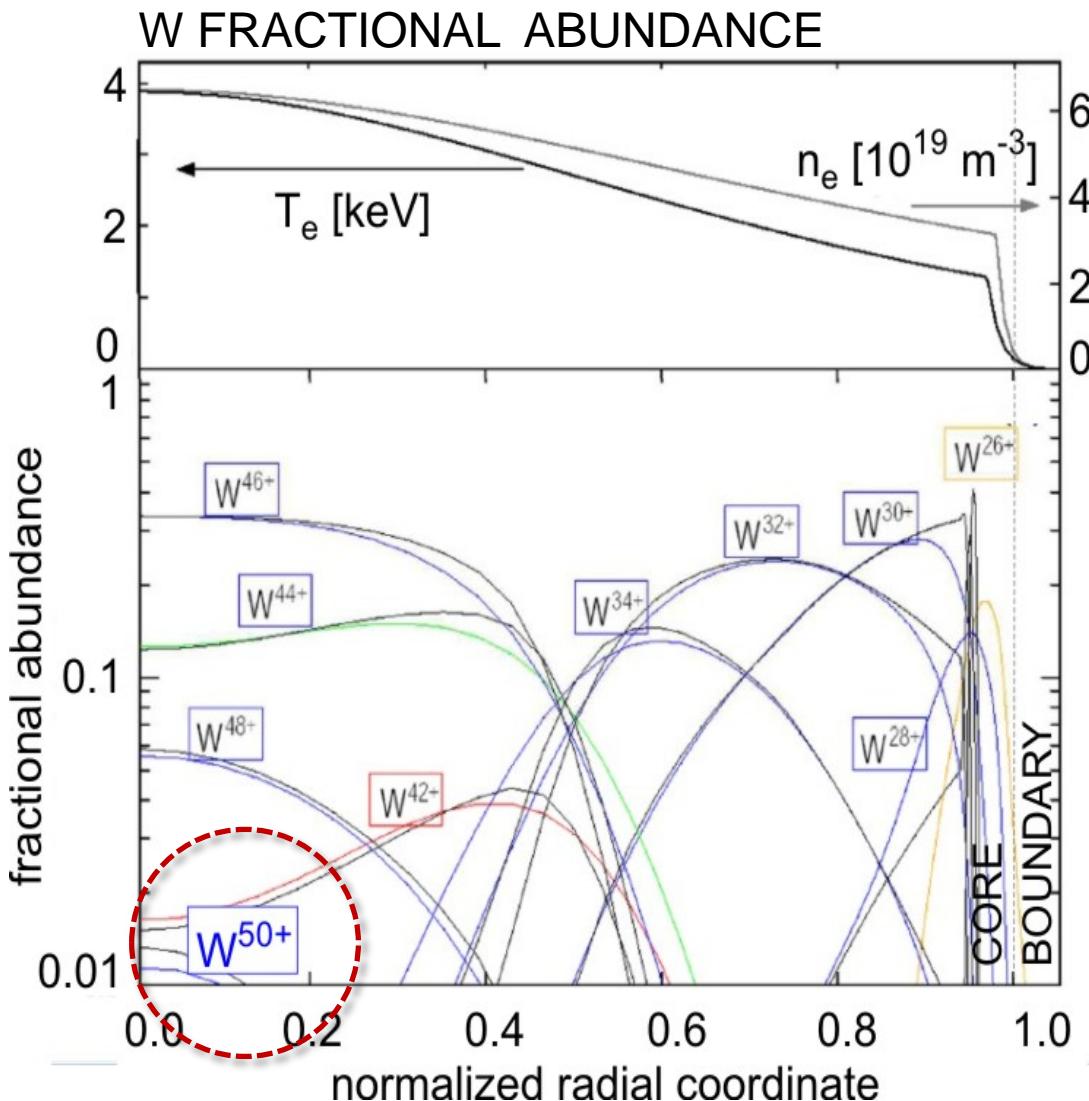
ionization/recombination/CX.
Atomic data models for hot plasma spectroscopy
• interpretation,
• line shape modelling:

Spectroscopy : nZ^*
CR Model : $nZ^* \rightarrow nZ$
Transport Model : $nZ \rightarrow D_{\perp}, V_{\perp}$

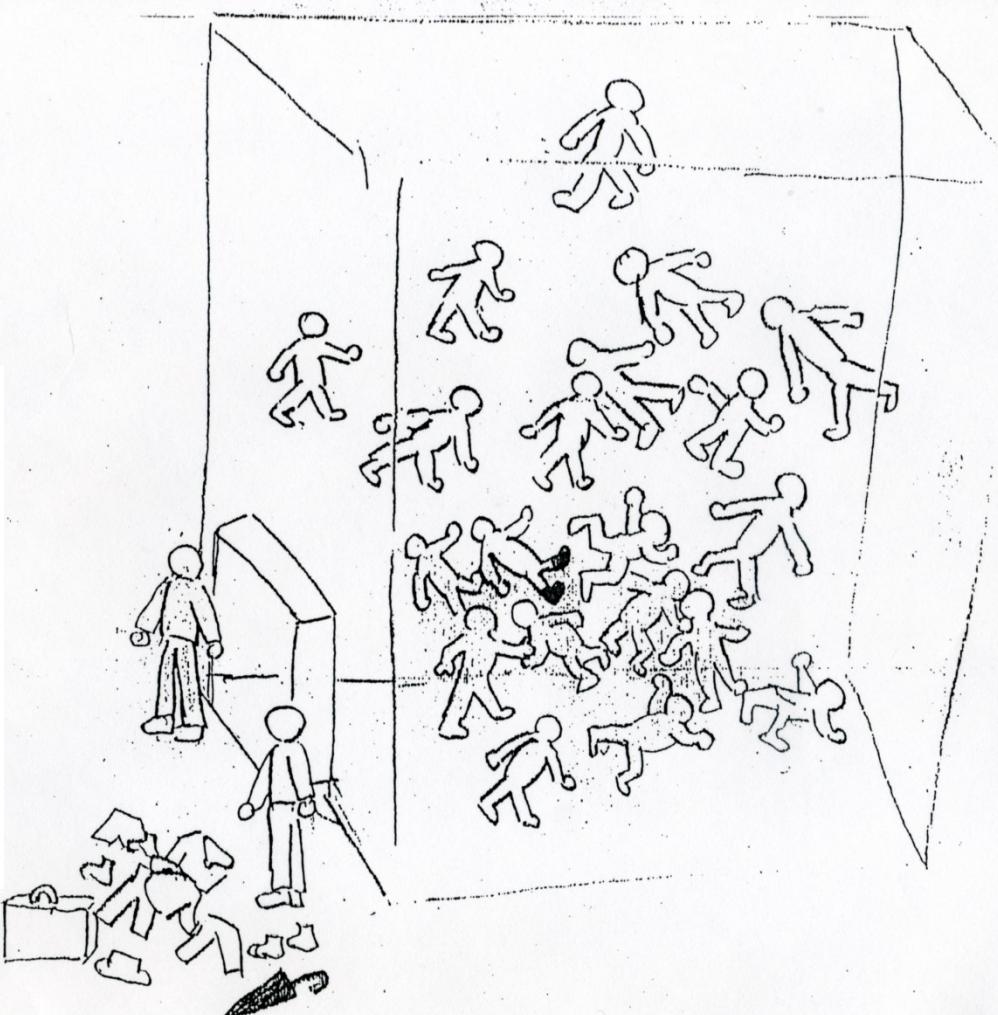
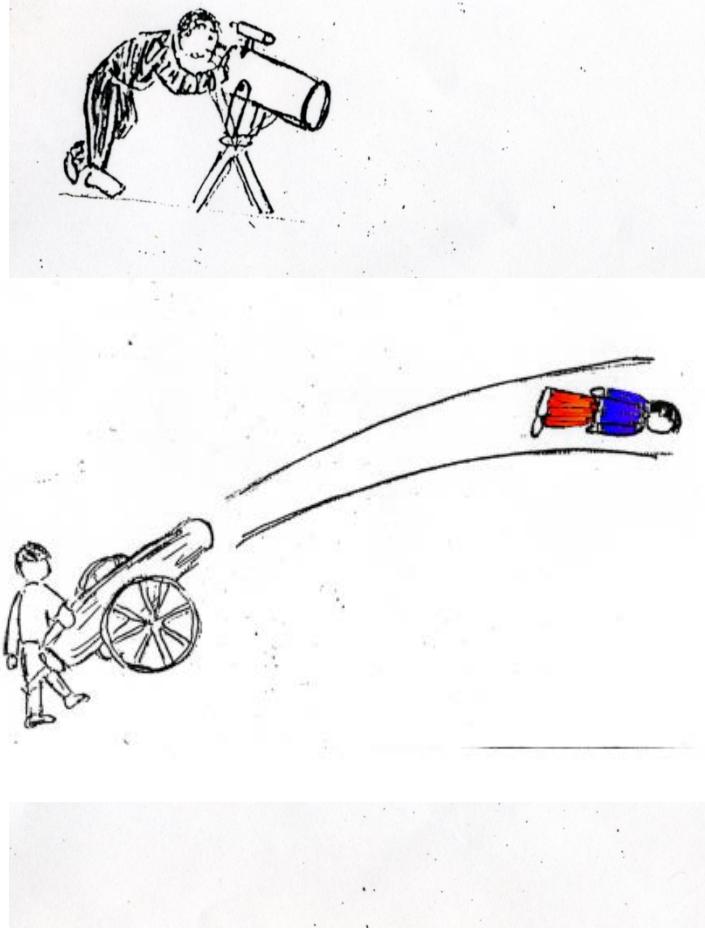
W atomic data development requested

Understanding W behaviour in the plasma core is very important for ITER and future high-performance, high-duty cycle devices

- W is a strong radiator at fusion core energies → concentration must be very low (< 0.001% in ITER)

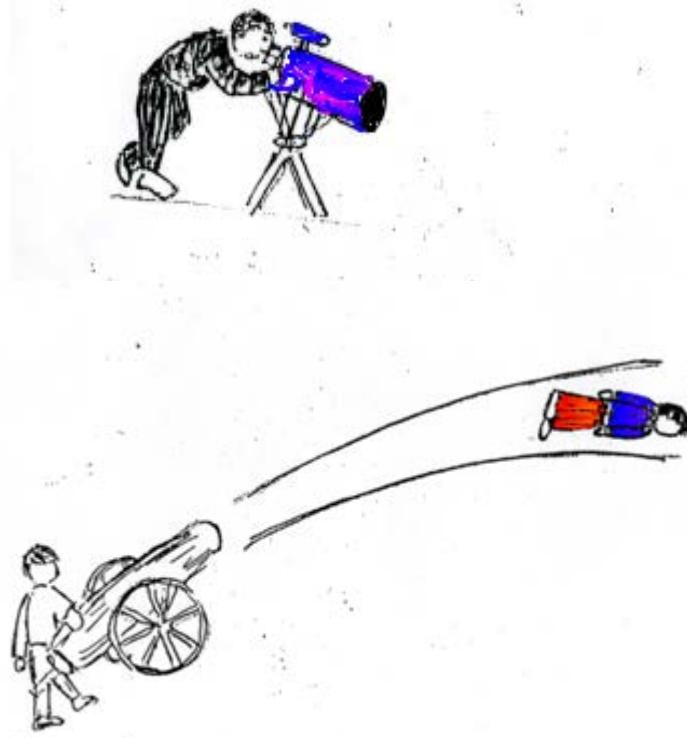


All relevant elements other than W are fully stripped of their electrons, except in the edge region

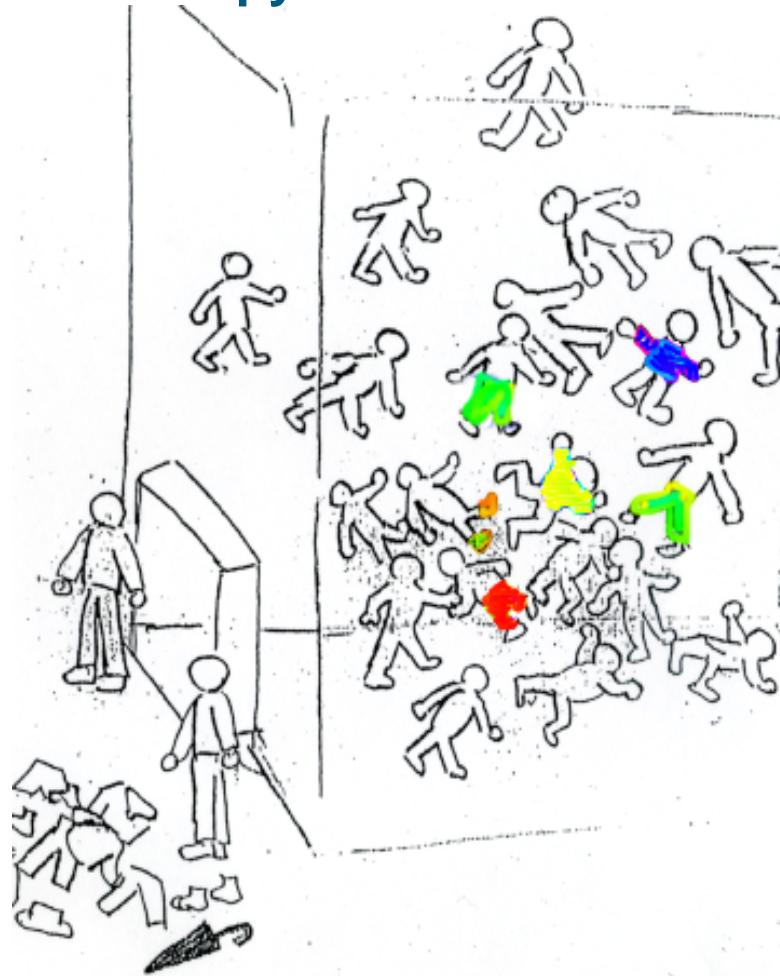


In the late 1970's Afrosimov suggested to inject high energy neutrals (hydrogen) which can pass through the magnetic field, primarily to heat the plasma by momentum transfer, but more importantly, passing electrons to fully stripped plasma ions.

Neutral (H) Beam (100 keV) injection provides electrons to elements, so they can be distinguished by spectroscopy



ITER NP diagnostic beam (India)
100 -1000 KeV



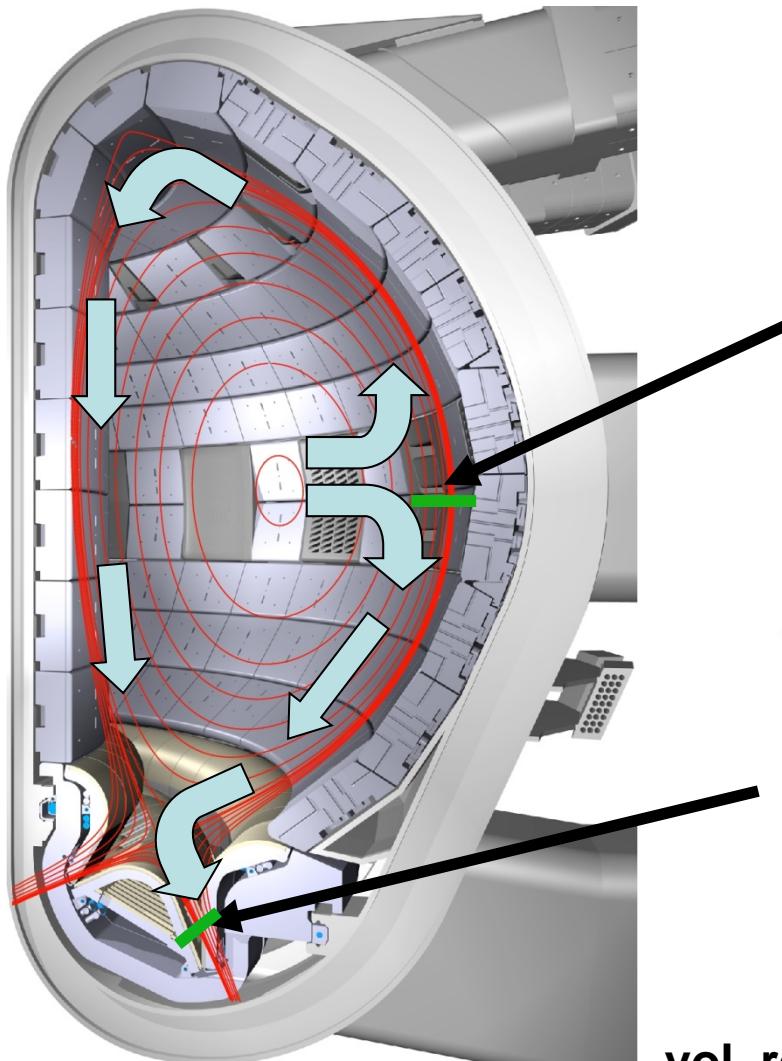
All elements (cooling gases, wall materials) become subject to plasma spectroscopy in fusion devices, because they are made “distinguishable” by H-diagnostic beams.

Outline

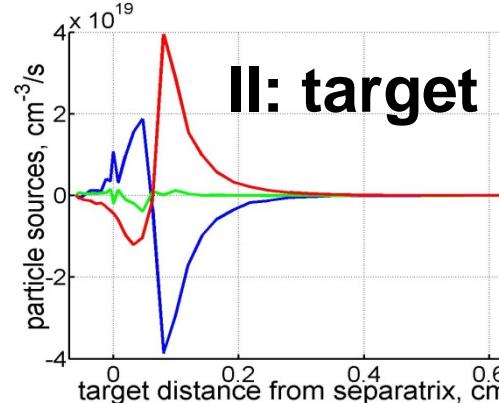
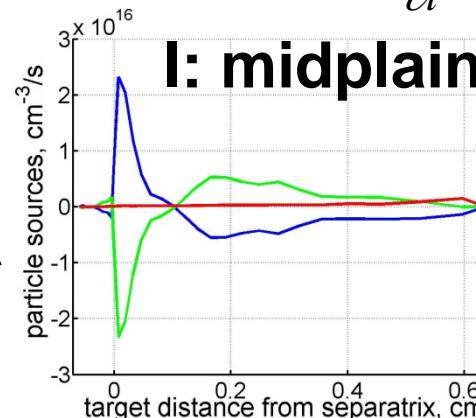
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Relative importance of plasma flow forces over chemistry and PWI: I edge region → II divertor

$$\underline{\text{div}(nv_{\parallel})} + \underline{\text{div}(nv_{\perp})} = \underline{\text{ionization/recombination/charge exchange}}$$



$$\frac{\partial}{\partial t} n_i + \vec{\nabla} \cdot (n_i \vec{V}_i) = S_{n_i}$$

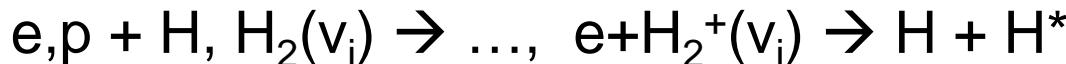


H, He, Be, W,... dominant process:
vol. recomb., & friction: p + H₂, “detachment”

parallel vs.
(turbulent)
cross field
flow

parallel vs.
chemistry
and PWI
driven flow

Atomic & molecular processes: boundary plasma



- divertor detachment dynamics, final states? Isotopes?

www.amdis.iaea.org, database & data center network,



R. Janev et al., Phys. Plasma (2004) 11, www.hydkin.de, IAEA: www.amdis.iaea.org



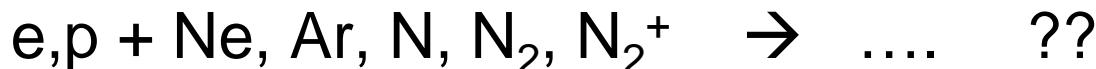
- H_3^+ probably irrelevant in fusion plasmas

M. Larsson et al., PRL (1993) 70, S. Datz et al., PRL (1995) 74, and: Conference series: DR 1-9



J.B. Roos et al. Phys. Rev A (2009) 80, IAEA Atomic Molec. data unit CRP 2012-2015

Exp.: UC Louvain, Theory: I. Schneider et al., Univ. Du Havre, J. Tennyson et al. (Quantemol), R. Celiberto et al. (Bari)

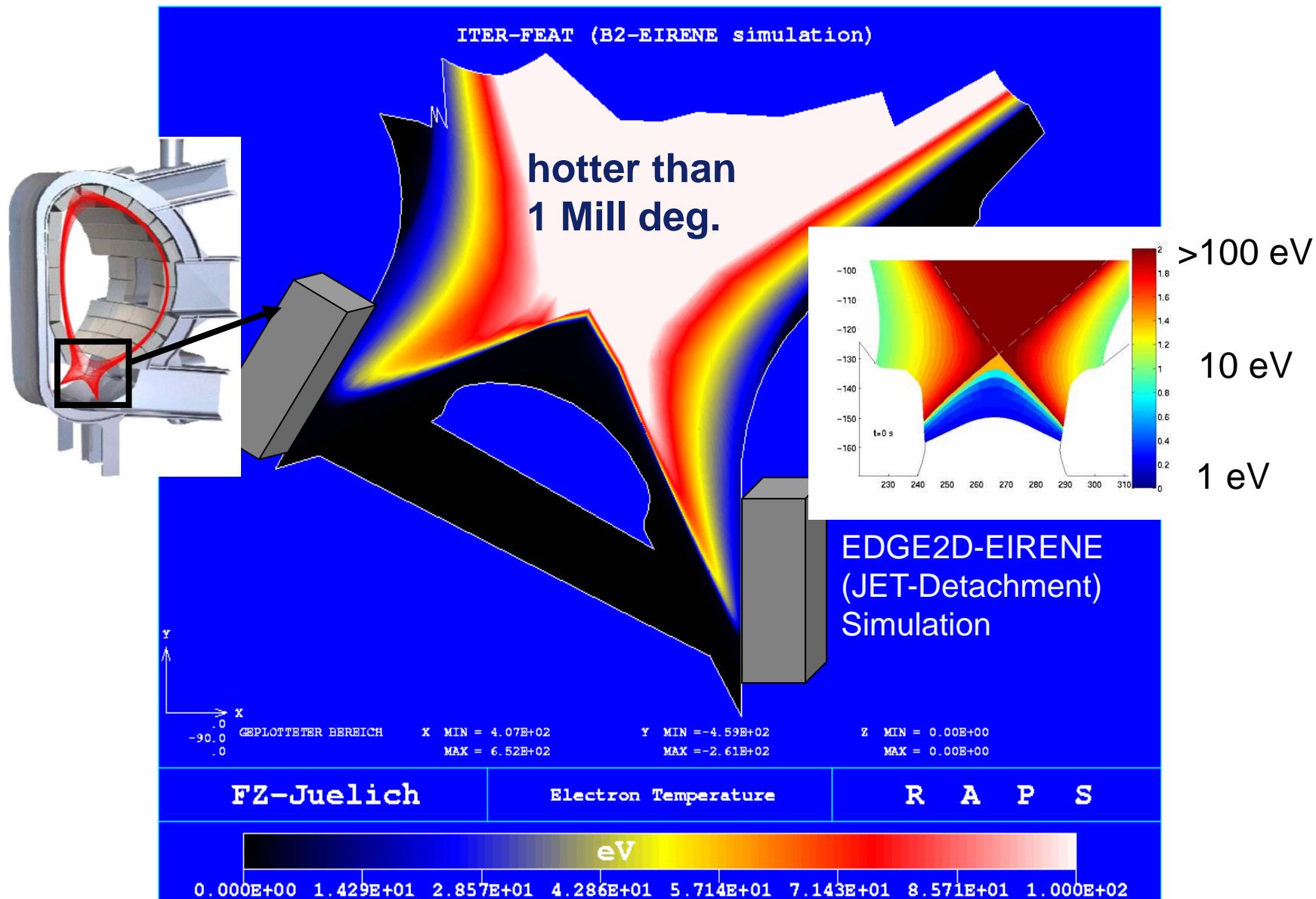


- N_2 -seeding, edge plasma cooling: molecular effects not yet studied in fusion plasmas, only resulting atomic ions N, N⁺, N⁺⁺,....

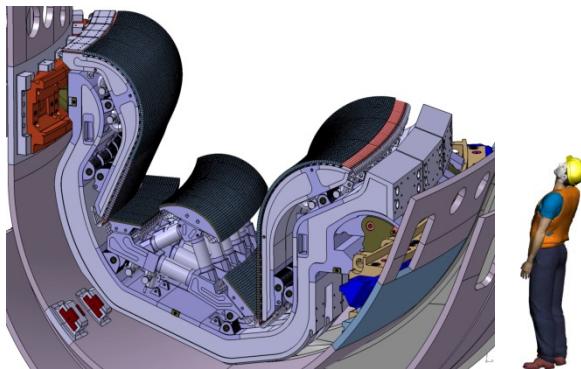
Extensive database exists, See planetary atmospheric entries research, e.g. A. Bultel et al, Universite de Rouen, France

Divertor detachment: ITER, simulation, detached,

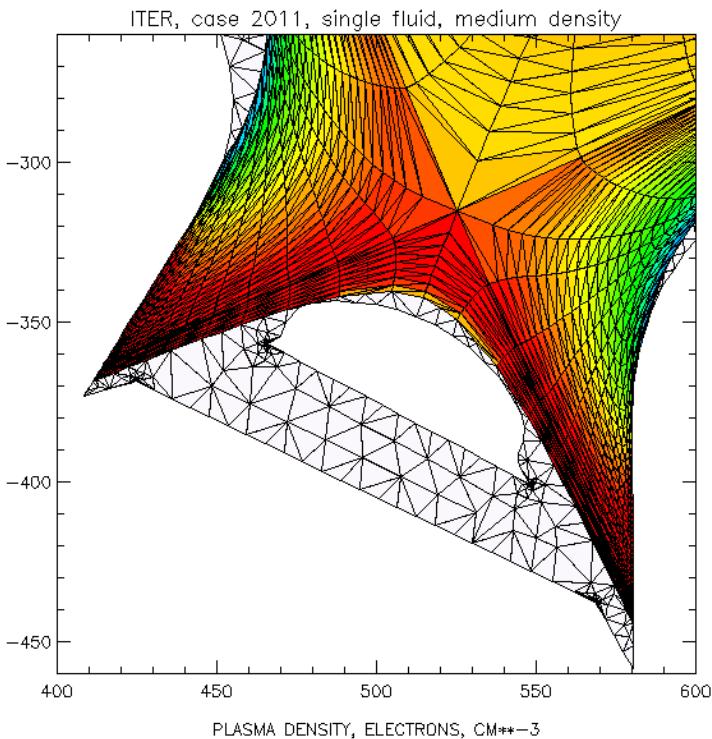
T_e field. Movie (JET): T_e during n_e ramp up, transition to detachment.



Narrowing down on the divertor plasma



Divertor plasma density
 cm^{-3} , log scale, 10^{12} - 10^{14}

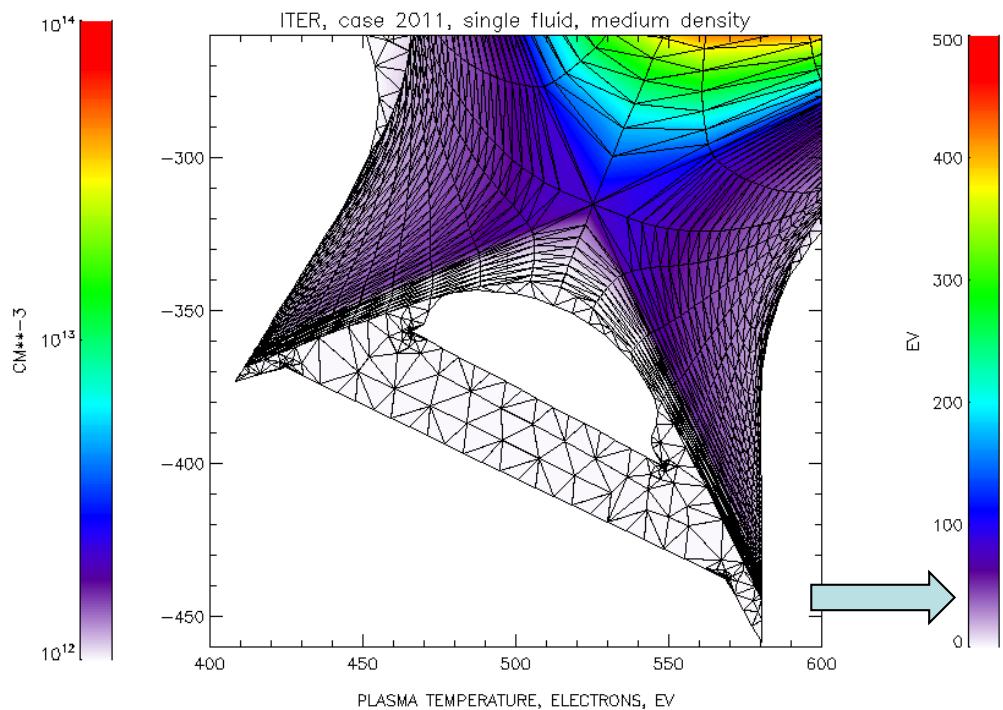


Electron thermalization time

$$\tau_{ee} \approx 3.3 \times 10^{-13} \left(\frac{T_e}{100 \text{ eV}} \right)^{3/2} \left(\frac{10^{21} \text{ cm}^{-3}}{n_i \log \Lambda} \right) \text{ s}$$

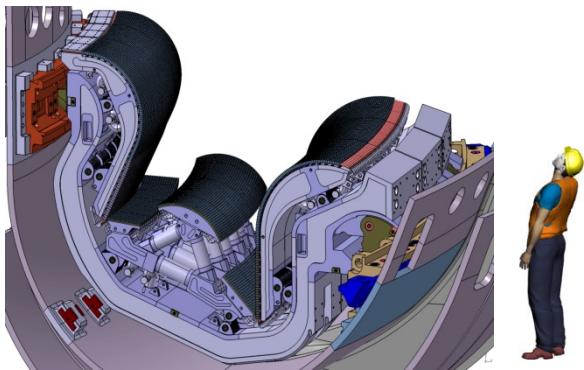
- Electrons well thermalized in divertor
- no need for EEDF considerations there

Divertor electron temperature
eV, lin. scale, 0 – 500 eV

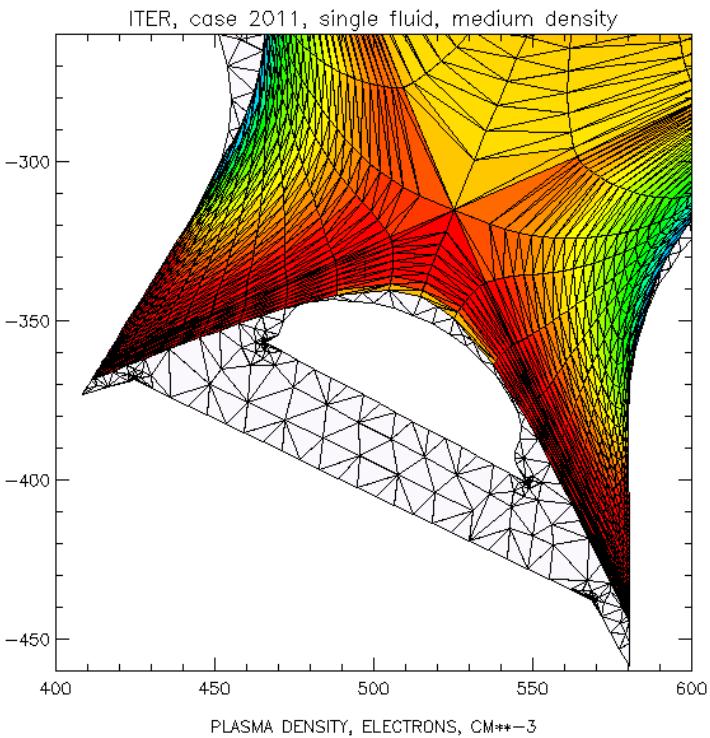


Plasma in “weakly detached” ITER divertor

$n_e: > 1e14, T_e < 5 \text{ eV}$



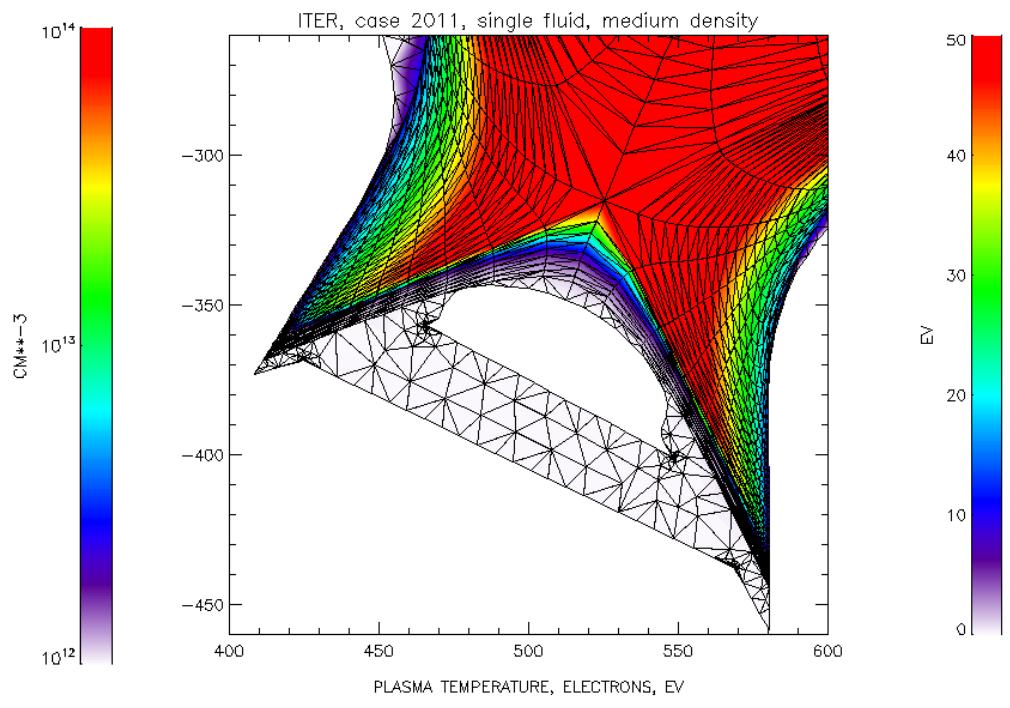
Divertor plasma density
 cm^{-3} , log scale, $10^{12} - 10^{14}$



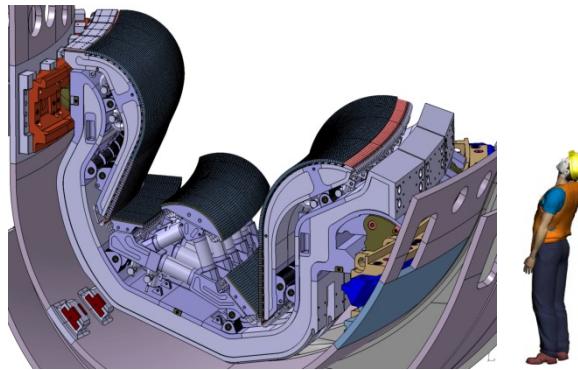
Tokamak Divertor Detachment:

- Self sustained dense, cold plasma layer ($\approx 1 - 3 \text{ eV}$) formed in front of high heat flux components.
- Plasma flux drops, despite increased density

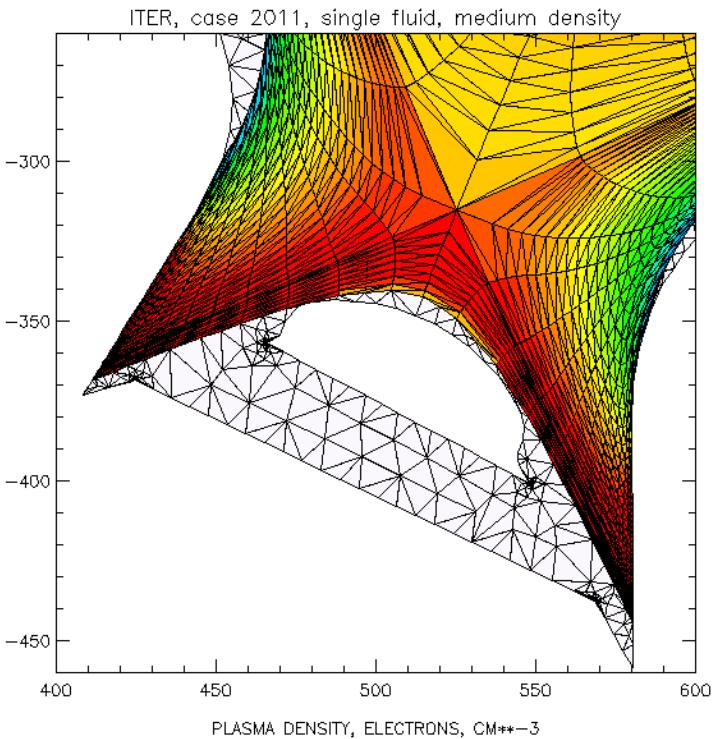
Divertor electron temperature
eV, lin. Scale, 0 – 50 eV



Electron – ion temperature equilibration: → Divertor $T_i \approx T_e$



Divertor plasma density
 cm^{-3} , log scale, 10^{12} - 10^{14}

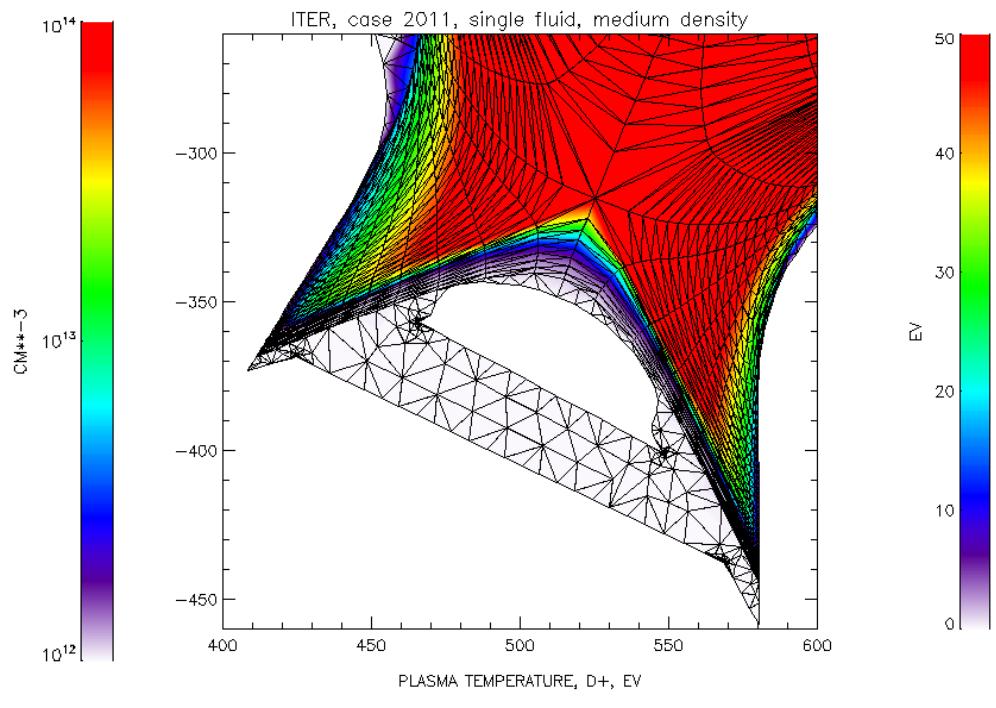


$T_e - T_i$ equilibration time

$$\tau_{eq} \approx 3.16 \times 10^{-10} \frac{A}{Z^2} \left(\frac{T_e}{100 \text{ eV}} \right)^{3/2} \left(\frac{10^{21} \text{ cm}^{-3}}{n_i \log \Lambda} \right) \text{ s}$$

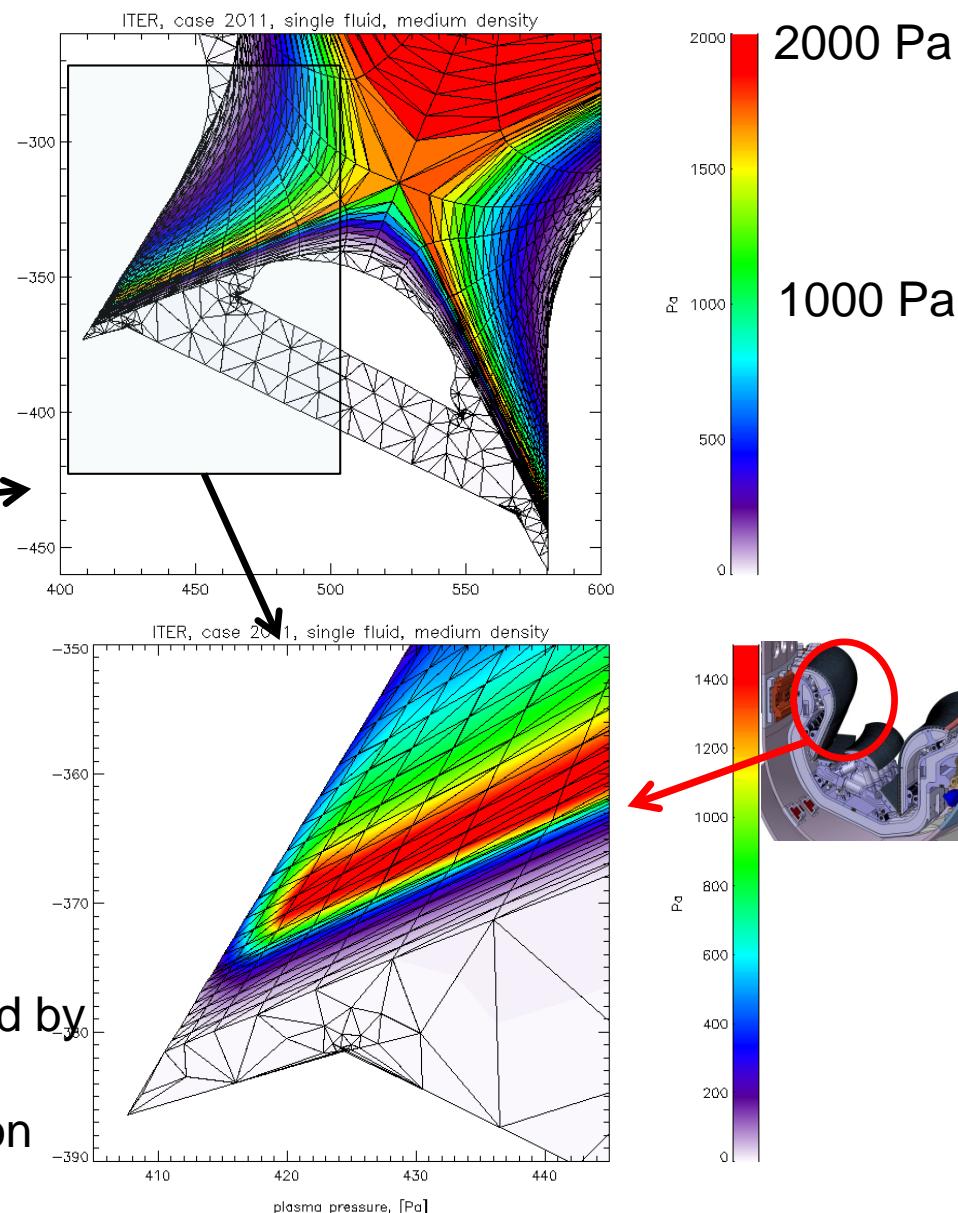
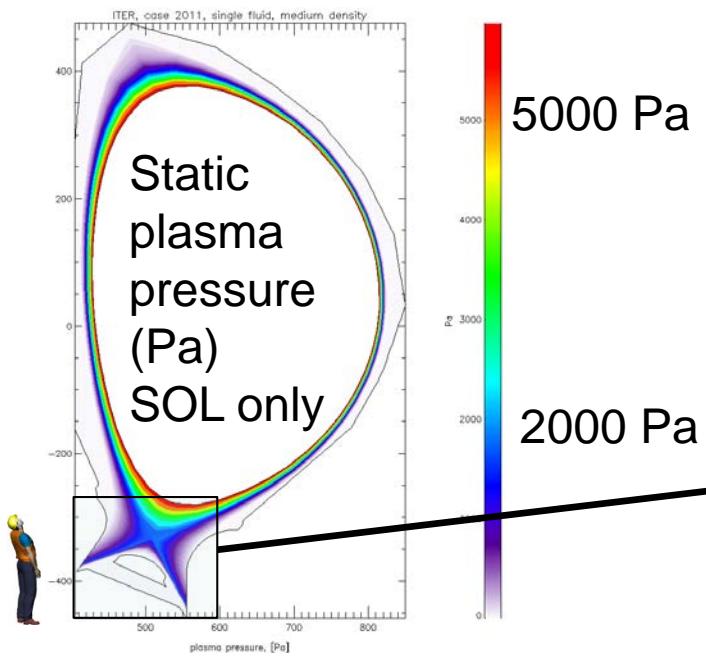
$\approx 1000 \times \tau_{ee}$ (for H plasma), but still fast.

Divertor D⁺ ion temperature
eV, lin. scale, 0 – 50 eV



(static) plasma pressure (Pa)

- Inside separatrix (confined plasma): constant on magn. flux surf.
- In divertor: pressure drop along B-field



Momentum balance for H⁺ ions

$$\frac{\partial}{\partial t} (m_i n_i \vec{V}_i) + \vec{\nabla} \cdot (m_i n_i \vec{V}_i \vec{V}_i) =$$

$-\vec{\nabla} p_i - \vec{\nabla} \cdot \vec{\Pi}_i + Z_i e n_i (\vec{E} + \vec{V}_i \times \vec{B}) + \vec{R}_i + \vec{S}_{m_i \vec{V}_i}$

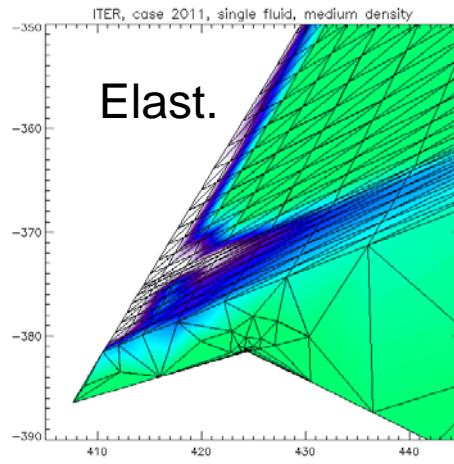
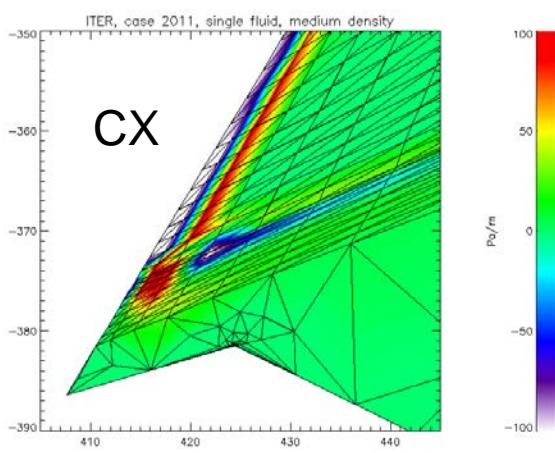
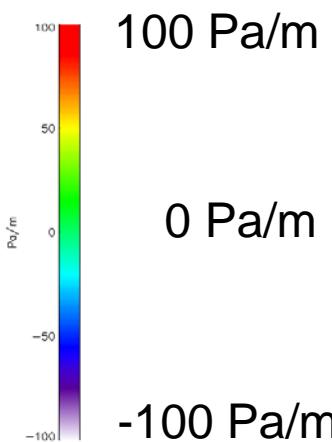
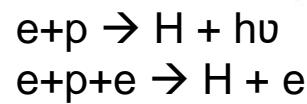
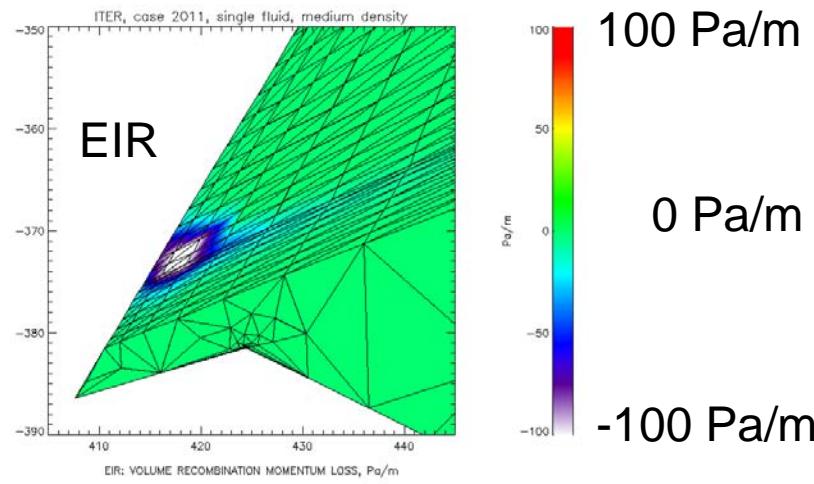
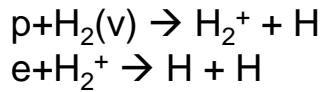
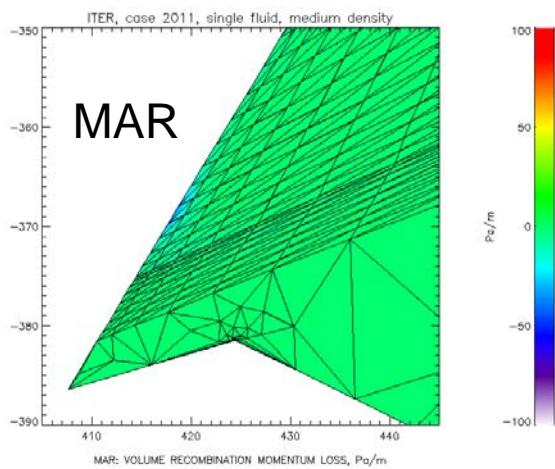
Near target pressure gradient balanced by momentum sinks:

- e + H⁺ plasma volume recombination
- Neutral gas – plasma friction

ITER divertor detachment:

Plasma pressure gradient [Pa/m] provided by:

MAR < EIR < $p+H$ (CX) < $p+H_2$ (elastic) friction



Plasma chemistry is localized in divertor.
Provides powerful particle, momentum and energy volumetric sources for plasma flow.

Continuity eq. for ions and electrons

$$\frac{\partial}{\partial t} n_i + \vec{\nabla} \cdot (n_i \vec{V}_i) = S_{n_i}$$

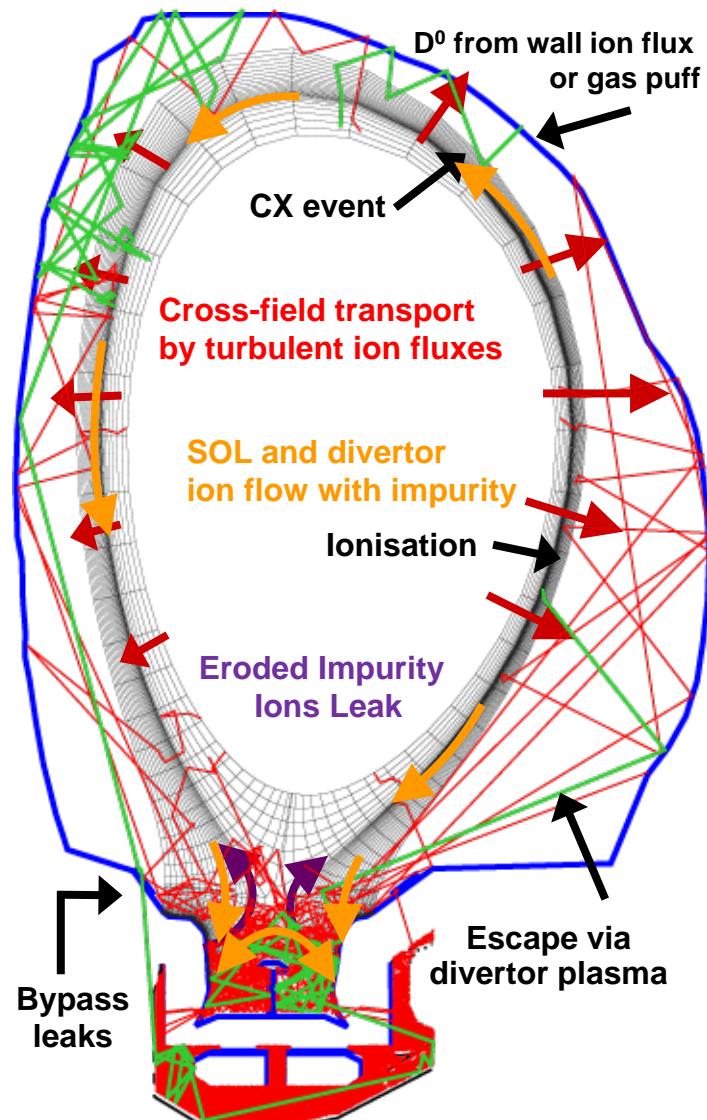
Momentum balance for ions and electrons

$$\begin{aligned} & \frac{\partial}{\partial t} (m_i n_i \vec{V}_i) + \vec{\nabla} \cdot (m_i n_i \vec{V}_i \vec{V}_i) = \\ & -\vec{\nabla} p_i - \vec{\nabla} \cdot \vec{\Pi}_i + Z_i e n_i (\vec{E} + \vec{V}_i \times \vec{B}) + \vec{R}_i + S_{m_i \vec{V}_i} \\ & -\vec{\nabla} p_e - e n_e (\vec{E} + \vec{V}_e \times \vec{B}) + \vec{R}_e = 0 \end{aligned}$$

energy balance for ions and electrons

$$\begin{aligned} & \frac{\partial}{\partial t} \left(\frac{3}{2} n_i T_i + \frac{m_i n_i}{2} \vec{V}_i^2 \right) + \\ & \vec{\nabla} \cdot \left[\left(\frac{5}{2} n_i T_i + \frac{m_i n_i}{2} \vec{V}_i^2 \right) \vec{V}_i + \vec{\Pi}_i \cdot \vec{V}_i + \vec{q}_i \right] \\ & = (e n_i Z_i \vec{E} - \vec{R}) \cdot \vec{V}_i - Q_{ei} + S_E^i \end{aligned}$$

$$\frac{\partial}{\partial t} \left(\frac{3}{2} n_e T_e \right) + \vec{\nabla} \cdot \left(\frac{5}{2} n_e T_e \vec{V}_e + \vec{q}_e \right) = -e n_e \vec{E} \cdot \vec{V}_e + \vec{R} \cdot \vec{V}_i + Q_{ei} + S_E^e$$



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In magnetic fusion:

focus is on plasma flow, turbulence,...,
(unknown, and computational challenge),
taking the

S terms (A&M data) **as „known“,**
and computationally „in hand“

Selection:

Non atomic physics experts
Disconnection between „fusion“ and
atomic physics communities

Implementation:

Very time consuming
Software duplication

Results:

Comparison of results from different
groups is often highly dependent
on the atomic data used

See: Enrico Landi (CHIANTI),
on atomic database for astrophysical
plasmas (IAEA-NFRI, 2012, Daejeon)



- divertor detachment dynamics, final states? DR, DE ?

H. Takagi, Phys. Scr. (2002) 52, and: Conference series: DR 1-9



R. Janev et al., Phys. Plasma (2004) 11



- H_3^+ probably irrelevant in fusion plasmas

M. Larsson et al., PRL (1993) 70, S. Datz et al. (1995) 74, and: Conference series: DR 1-9



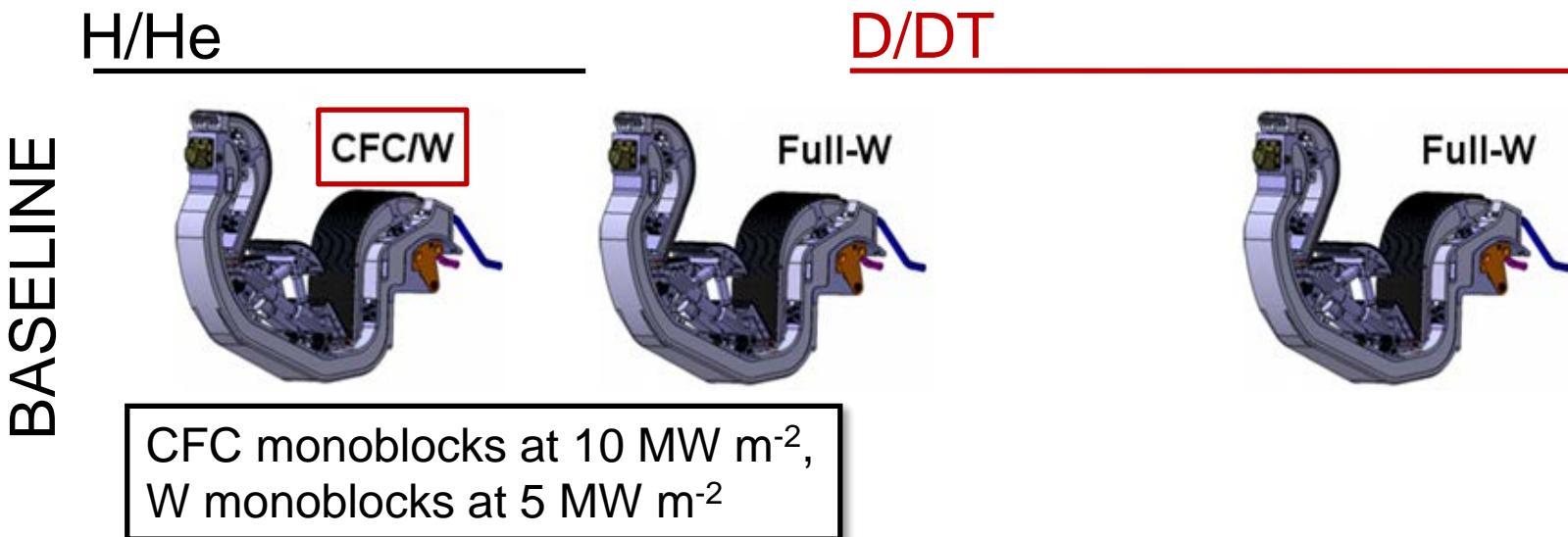
J.B. Roos et al. Phys. Rev A (2009) 80, IAEA Atomic Molec. data unit CRP 2012-201



- N_2 -seeding, edge plasma cooling: not yet studied in fusion plasmas, only resulting atomic ions N, N^+, N^{++}, \dots

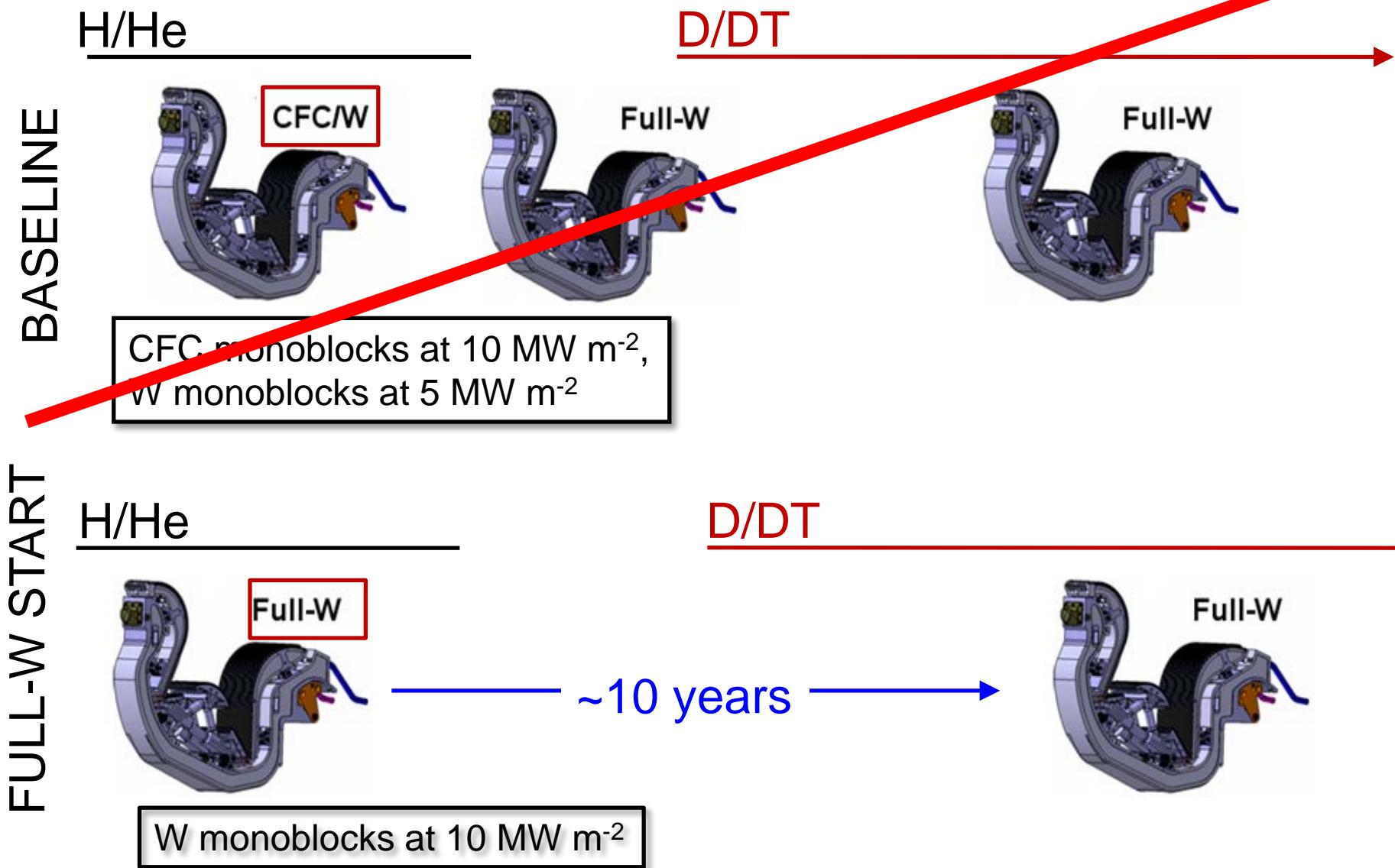
See planetary atmospheric research

Proposed ITER divertor strategy: decision at the end of 2013



21.11.2013; The ITER Council approved the IO proposal and decided to commence operations with a **full tungsten divertor...**

Proposed divertor strategy: decision at the end of 2013



Tokamak/Stellarator boundary: hybrid plasma (fluid) neutral/impurity (kinetic)

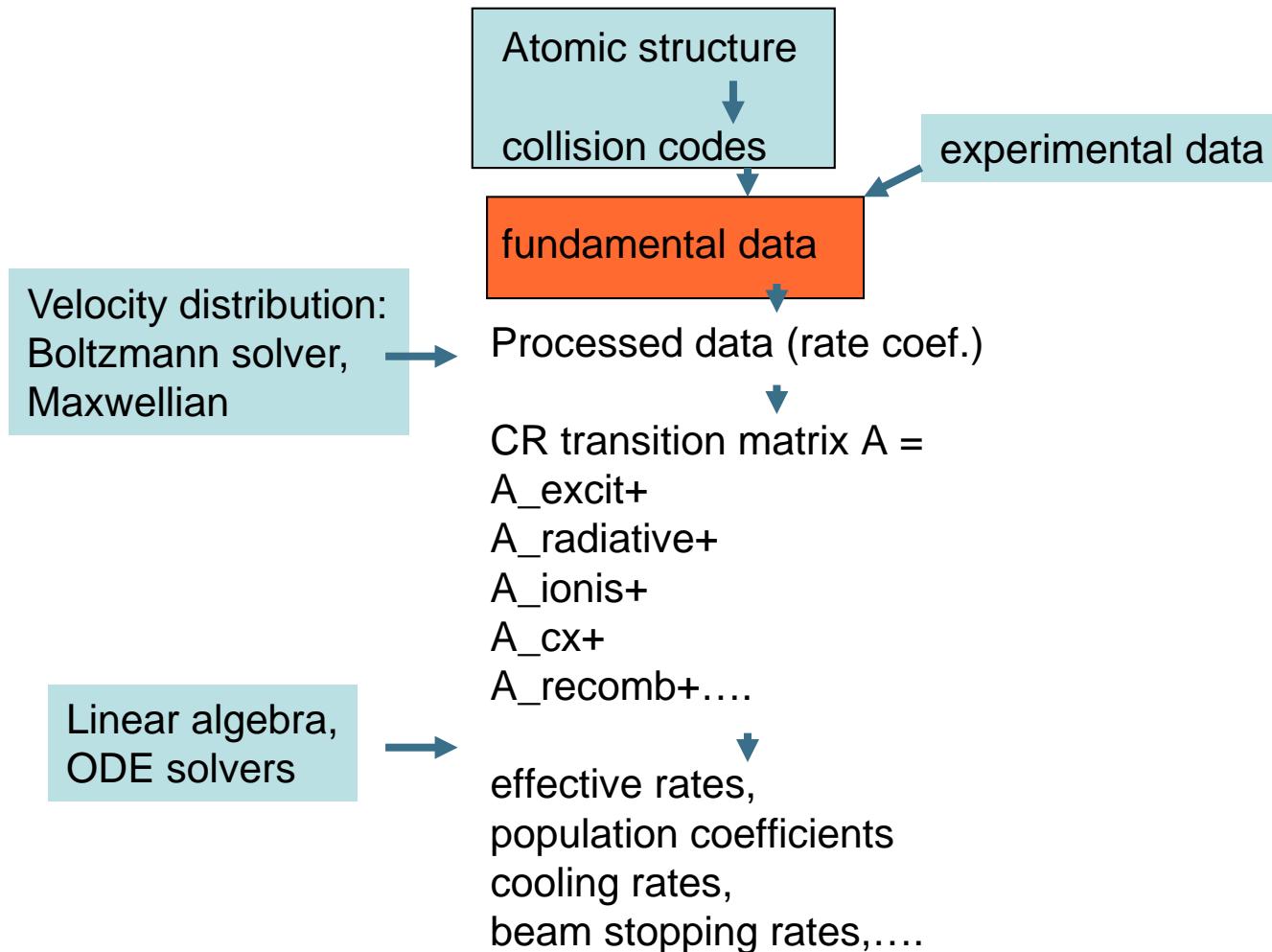
- **Status:**
transition from computational science to computational engineering currently ongoing (e.g. divertor design for ITER, DEMO, W7X, JT-60SA...) despite many deficits still
 - **But: long list of deficient understanding:**
In particular: plasma material interaction (empirical laws only)
but also: sources, parallel flows, cross field turbulent fluxes....
 - **Goal:** separate all known (ab initio) model parts from the still unknown (ad hoc) parts, often by detailed computational bookkeeping.
Ultimately: isolate anomalous cross field transport as only remaining unknown, to make it accessible experimentally.
- **sub-Goal: at least: turn A&M&S data issues from unkown to known (at least to: “evaluated”, “publicly exposed”, “ITER reference data set”)**

Thank you for your attention!

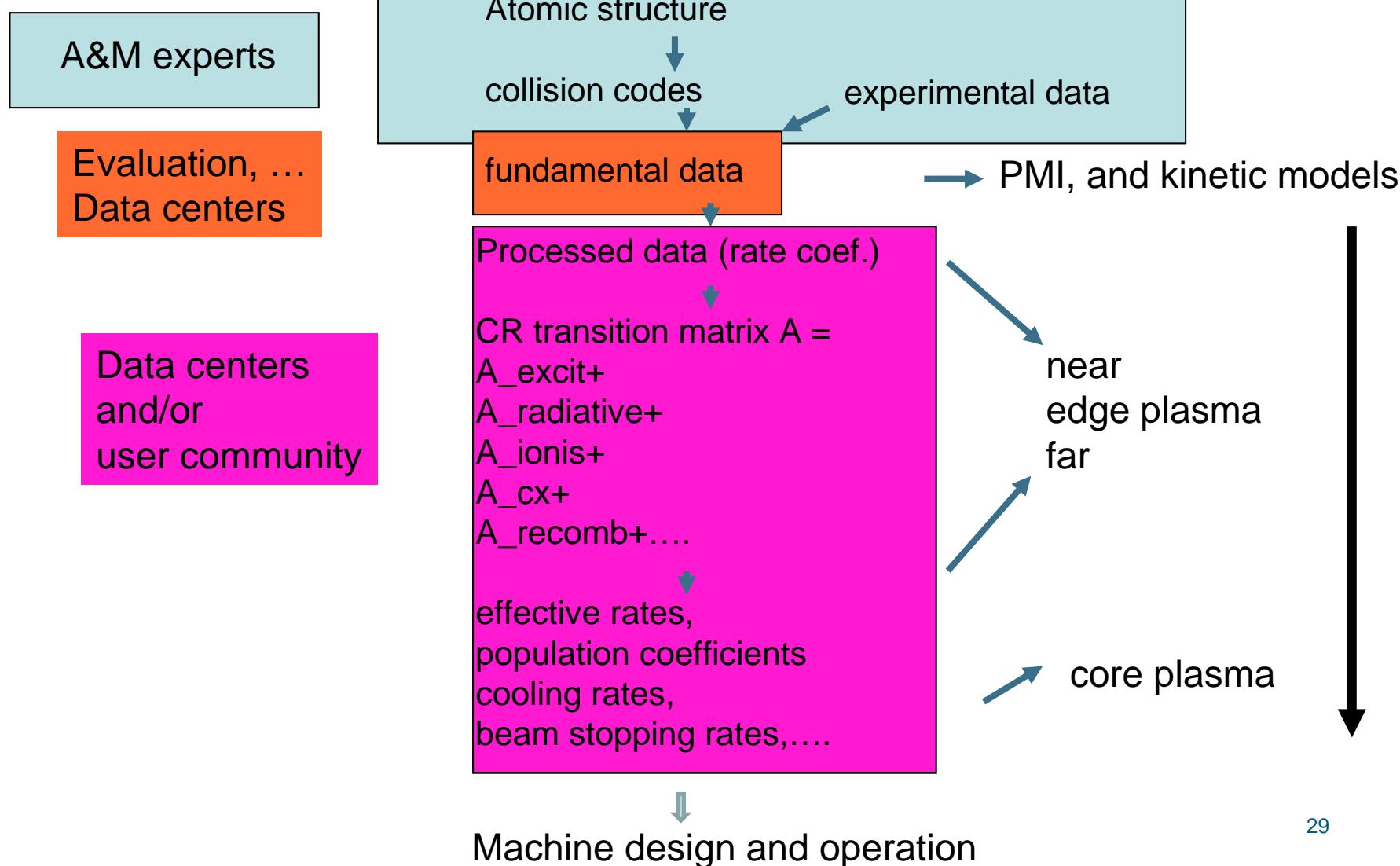
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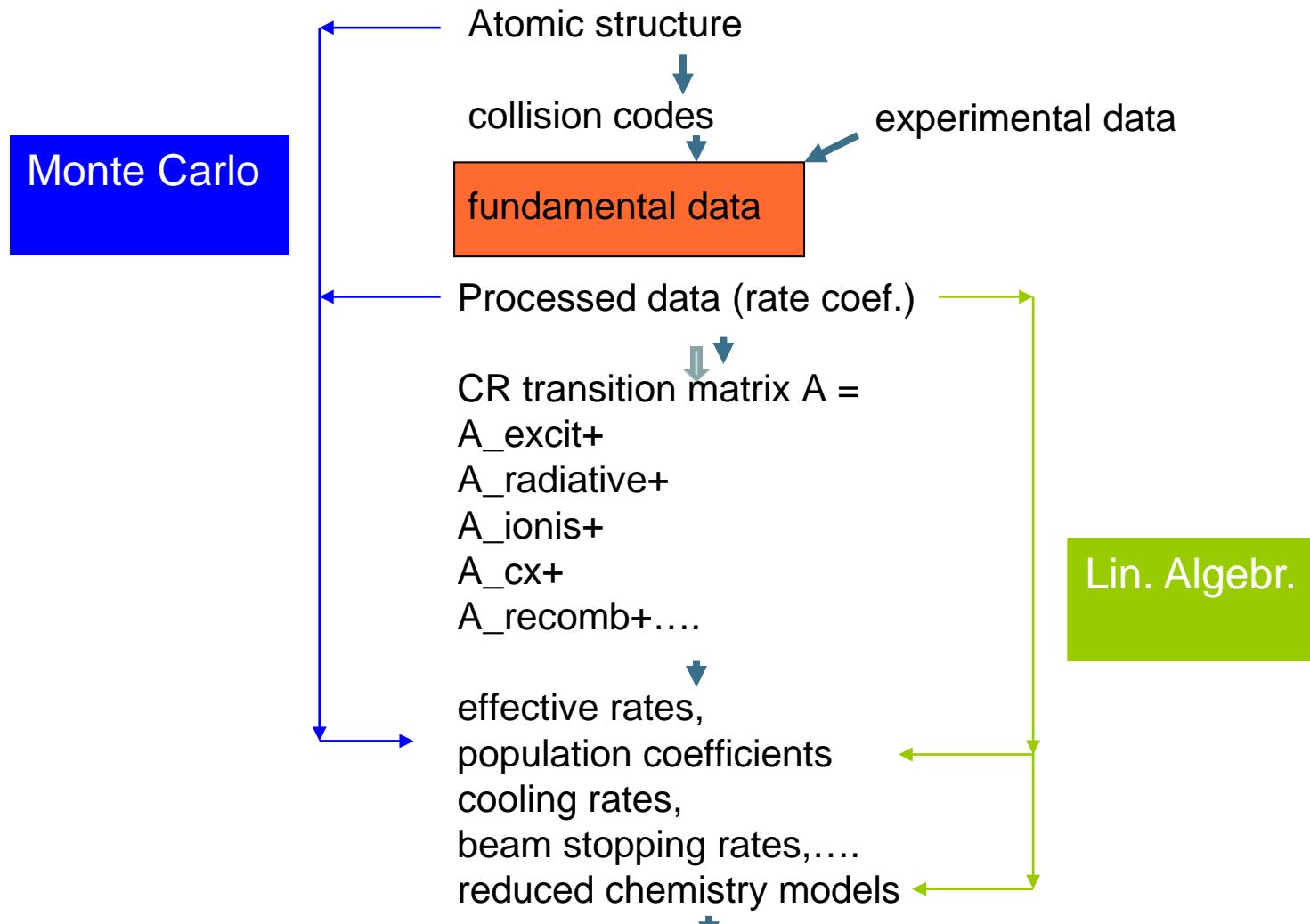
Uncertainty propagation and sensitivity analysis



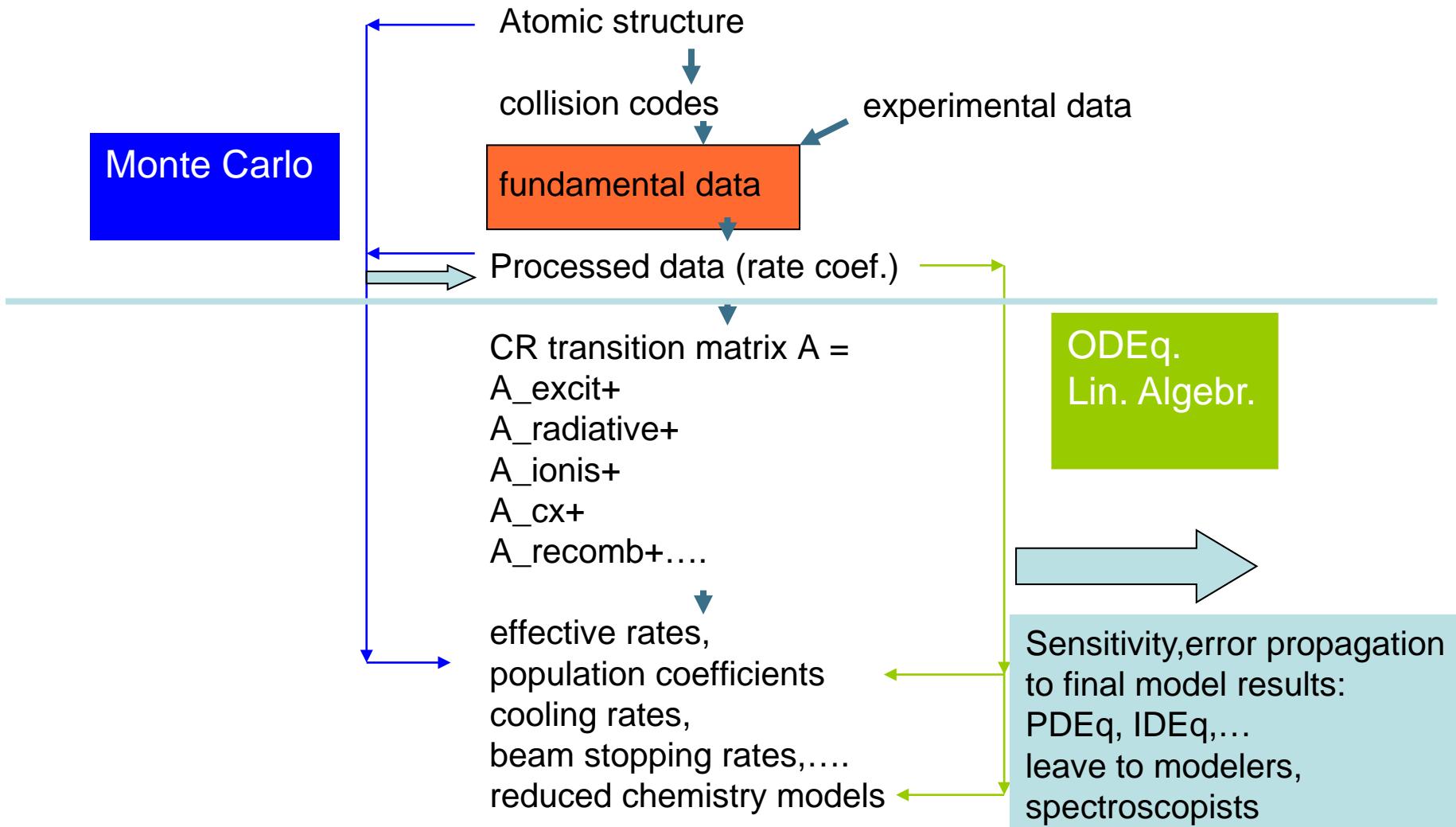
Uncertainty propagation and sensitivity analysis: journey of data...



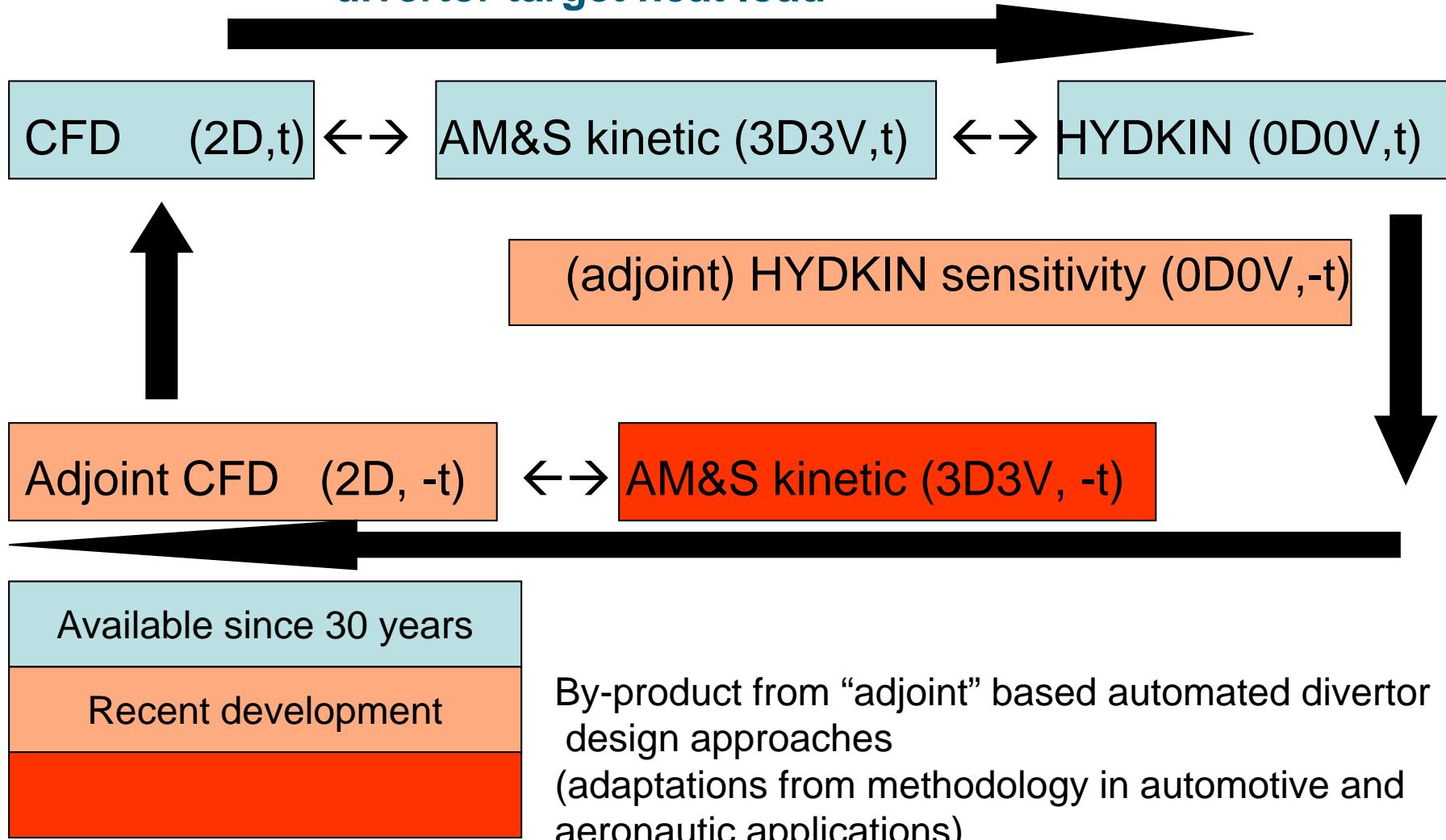
Uncertainty propagation and sensitivity analysis



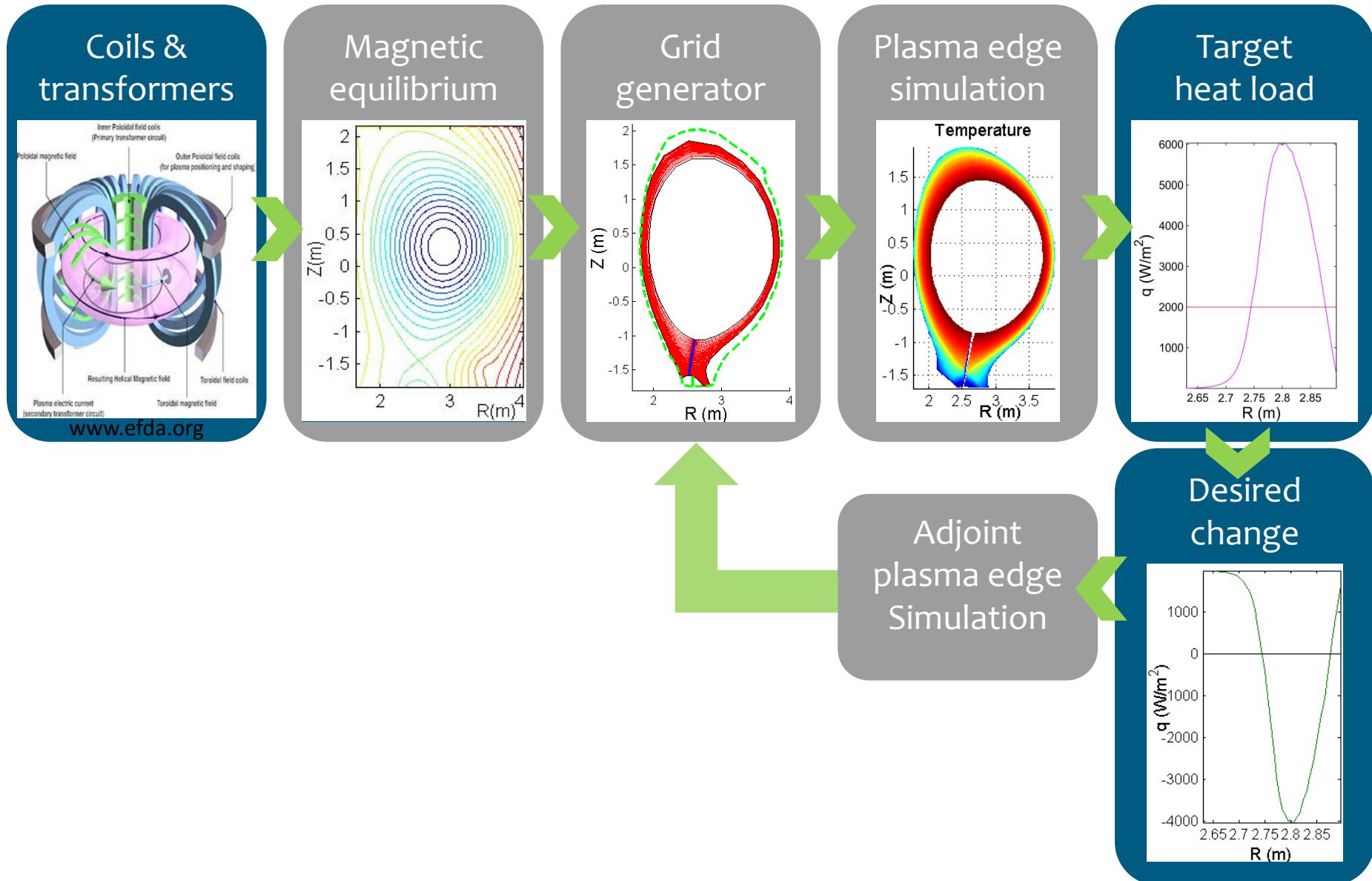
Uncertainty propagation and sensitivity analysis



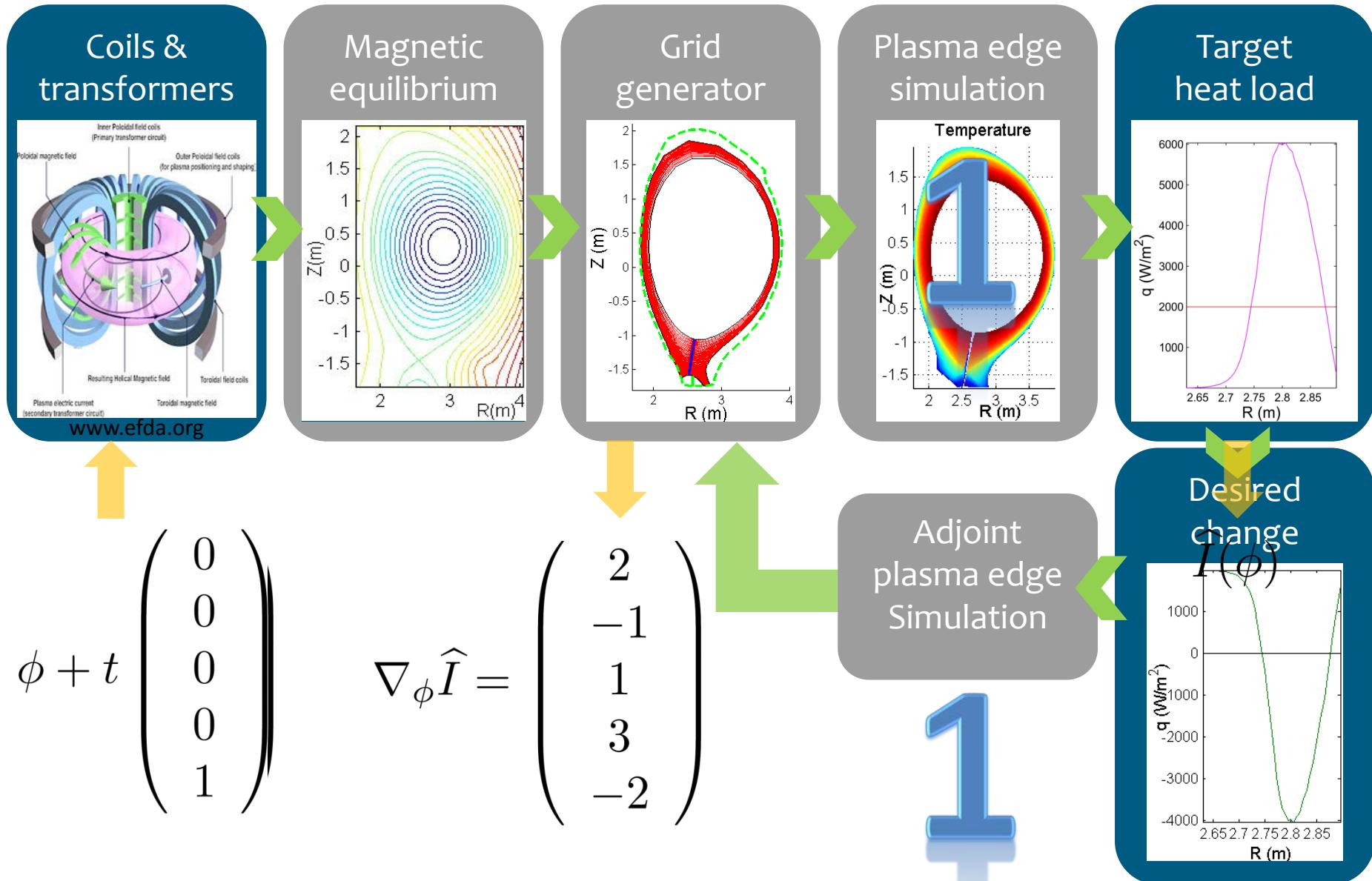
A&M sensitivity, uncertainty propagation in fusion reactor design and operation: How do uncertainties in A&M data propagate to divertor target heat load



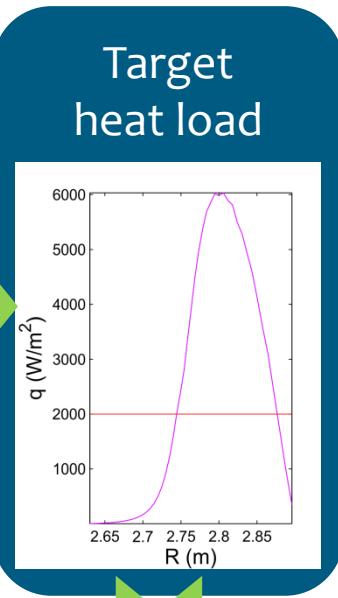
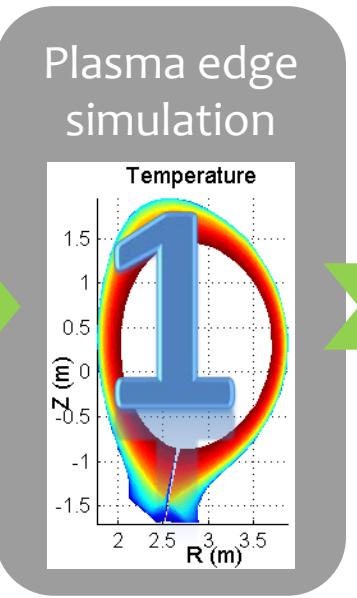
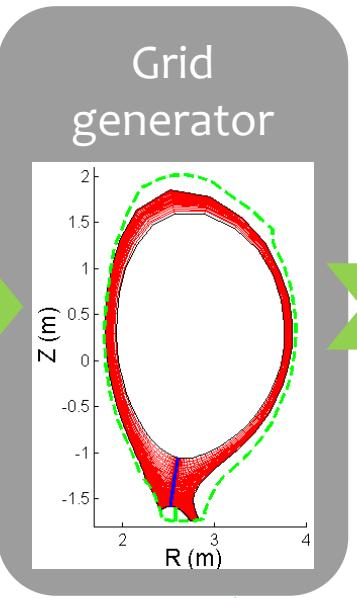
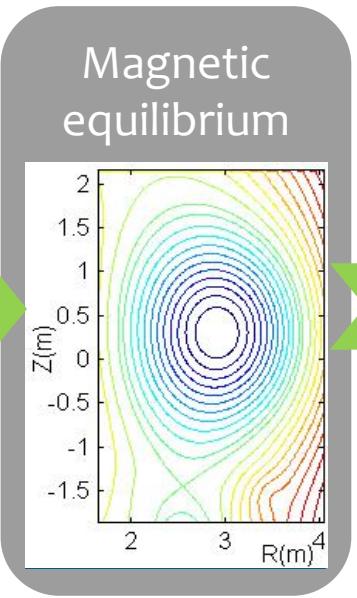
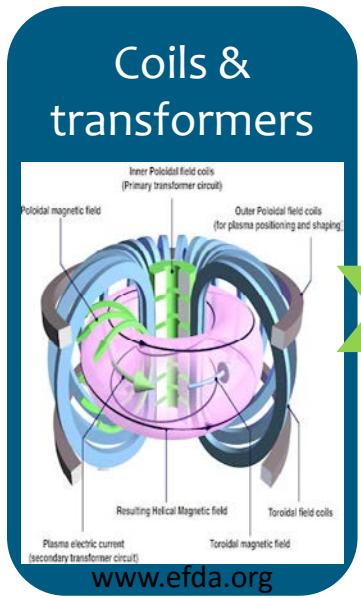
Introducing adjoint sensitivities



Sensitivities from adjoint flow simulation

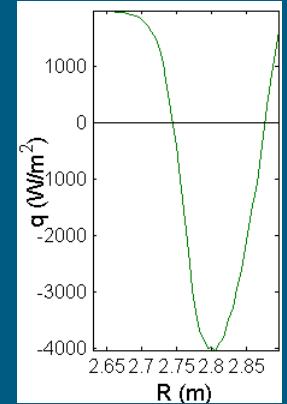


Sensitivities in practice



$$\text{cost}(\nabla_{\phi}\hat{I}) \approx 2 \text{ cost}(\hat{I})$$

Adjoint plasma edge Simulation



Details:

M. Blommaert et al. "A Novel Approach to Magnetic Divertor Configuration Design," Journal of Nuclear Materials, 2014.