

Preparatory Consultancy Meeting for the Injected Impurities, IAEA CRP

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Thanks for T.Oishi(NIFS), Z.S.Yang (ASIPP)

June 7, 2022, Remote meeting

Requests from IAEA-1

What data are needed for modelling the atomic and molecular physics of injected impurities?

Are impurities both for **power mitigation** and for **plasma diagnostics** in scope?

Which species should be in scope: certainly, N and Ne (maybe also Ar?)

for power control in the **divertor area**; Li, B and BN for edge plasma regions; others?

>>these materials are used for turbulence control (B) and impurity and hydrogen recycling control (B, Li)

*Method 1; pellet, 2; IPD, 3; gas,

4;Laser ablation (no page, for example SWIP in China)

Which processes are of greatest importance (collisions with H/D/T/He/e-, radiative line strengths), and which energy ranges (**1 – 100 eV up to several keV** for ELMs)?

What is the importance of molecules to the CRP, e.g. in the formation of NH_x isotopologues in the sub-divertor region?

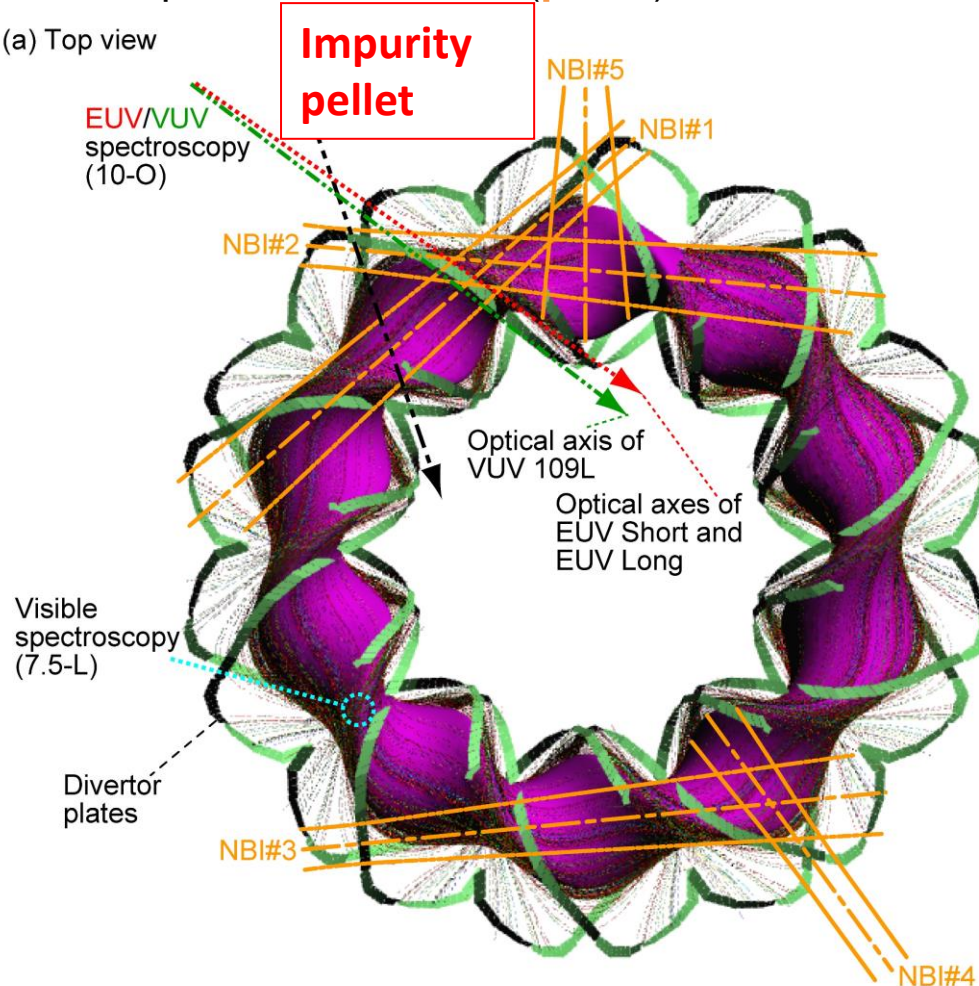
How are data incorporated into models like SOLPS-ITER and how can integration of new data be facilitated?

Method1; Spectroscopy and tungsten pellet injection in LHD

LHD

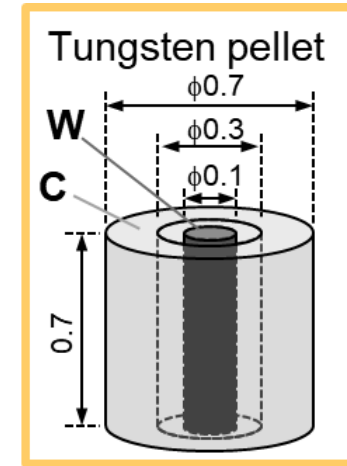
- $B_t < 3 \text{ T}$, $R = 3.6 \text{ m}$, $\langle a \rangle = 0.64 \text{ m}$
- Toroidal/poloidal period number = 10/2
- Heating: ECRH, NBI
 - negative ion source (**n-NBI**): NBI#1, #2, #3
 - positive ion source (**p-NBI**): NBI#4, #5

(a) Top view



Impurity pellet

- W wire in a C pellet:
10.7 mm x $\phi 0.1 \text{ mm}$
 $\sim 3.5 \times 10^{17} \text{ W atoms}$



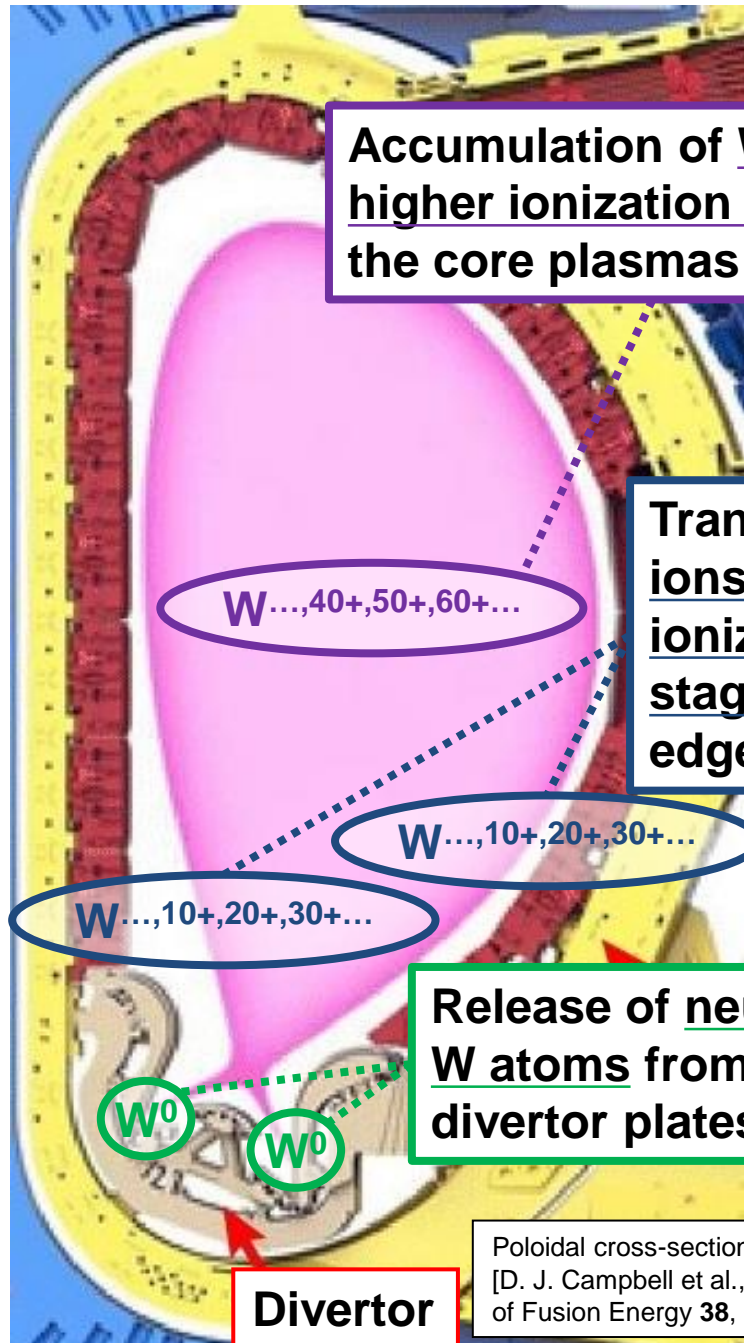
Spectroscopy

- Visible MK300: 3200–3550 Å
- VUV 109L: 250–1050 Å
- EUV Long: 100–300 Å
- EUV Short: 5–60 Å

LHD has two types of impurity pellet, “impurity pellet” and, “TESPEL” (Tracer-Encapsulated Solid Pellet)

Introduction: Tungsten behavior in fusion plasmas

T.Oishi



Accumulation of W ions at higher ionization stages in the core plasmas

Transport of W ions at lower ionization stages in the edge plasmas

Release of neutral W atoms from the divertor plates

Divertor

Poloidal cross-section of ITER
[D. J. Campbell et al., Journal of Fusion Energy **38**, 11 (2019)]

- **Tungsten** is regarded as a leading candidate material for the plasma facing component in ITER and future fusion reactors.
- **Spectroscopic studies** for tungsten impurity ions in magnetically-confined high-temperature plasmas

Neutral W atoms:
Visible spectroscopy

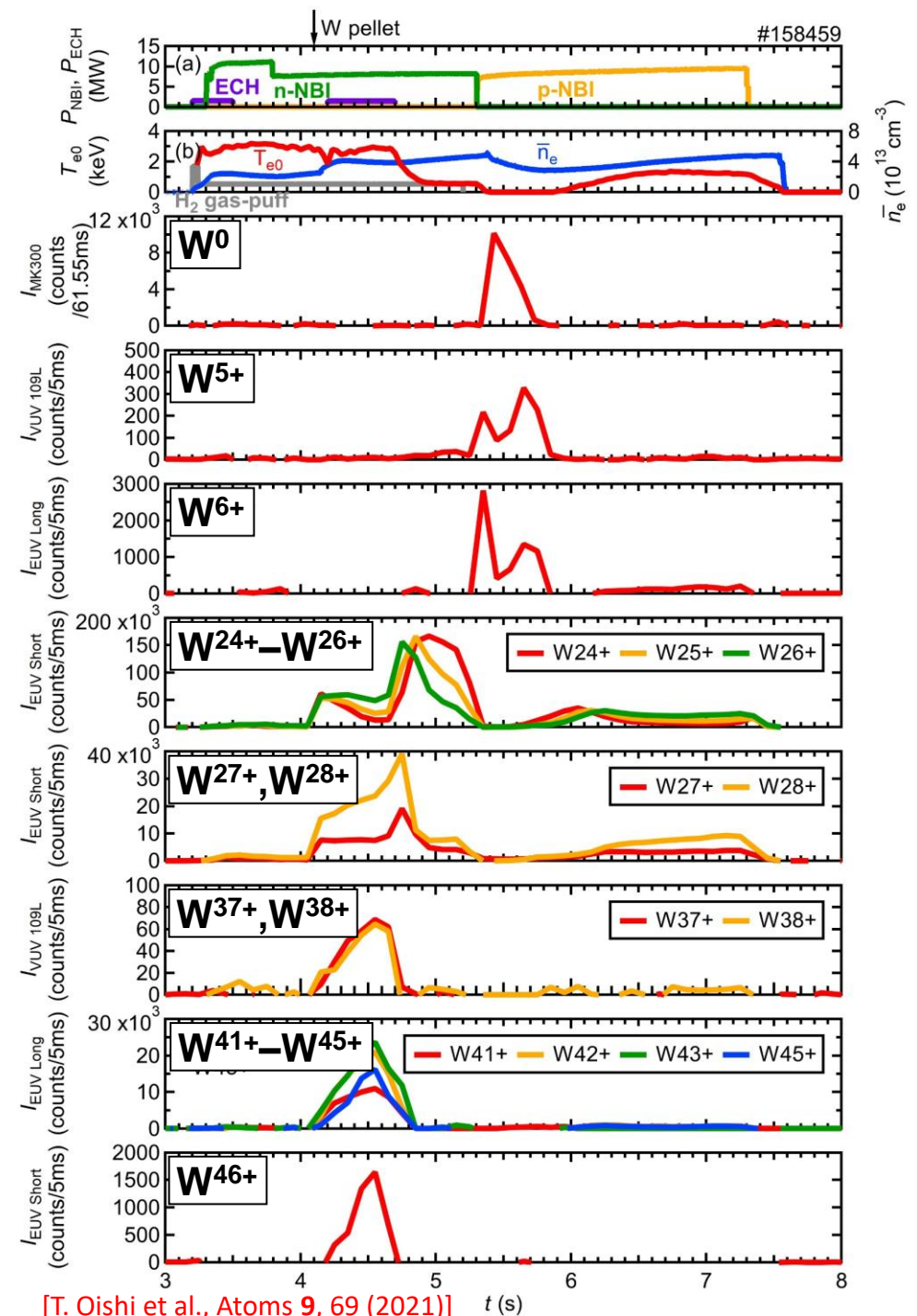
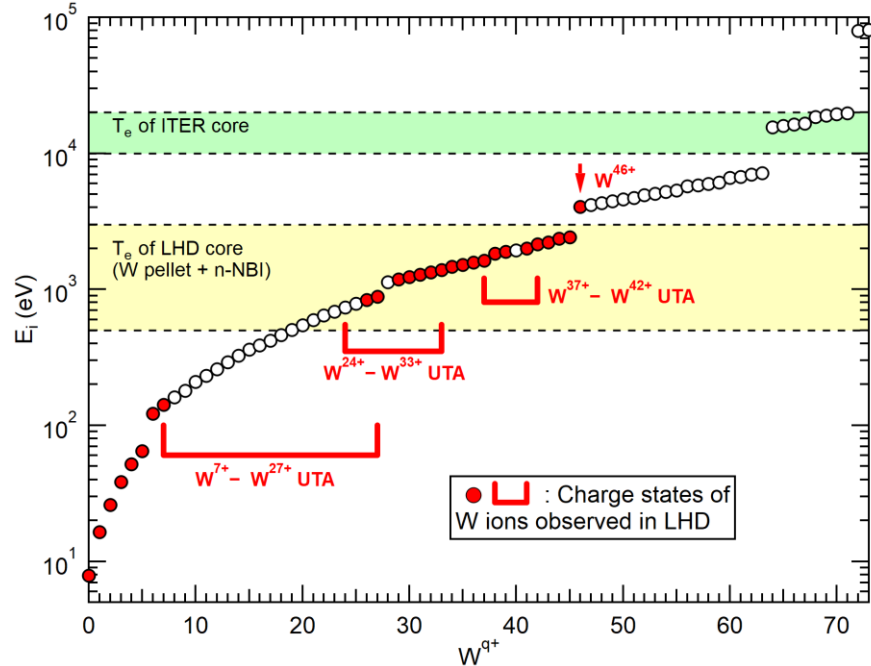
W ions at lower ionization stages:
VUV spectroscopy

W ions at higher ionization stages:
EUV spectroscopy, SX imaging

- In the Large Helical Device (LHD), spectroscopic studies for tungsten ions using a tungsten pellet injection technique have been intensively conducted.
- In this talk, the tungsten spectra in visible, VUV, and EUV wavelength ranges observed in LHD will be summarized.

Application of the observation (i) Temporal evolution of the W^0 – W^{46+} emission

- W^0 , W^{5+} , W^{6+} , W^{24+} – W^{28+} , W^{37+} , W^{38+} , and W^{41+} – W^{46+} line emissions were picked up as useful emission lines to monitor temporal evolution of emission intensity in a wide range of the charge states.
- The dominant charge state varied sequentially in time, together with T_{e0} . It is a reasonable relationship between T_e and the ionization energy, E_i .
- W^{10+} – W^{20+} emission data are insufficient.



T.Oishi

Method 2; Impurity powder dropper (IPD)

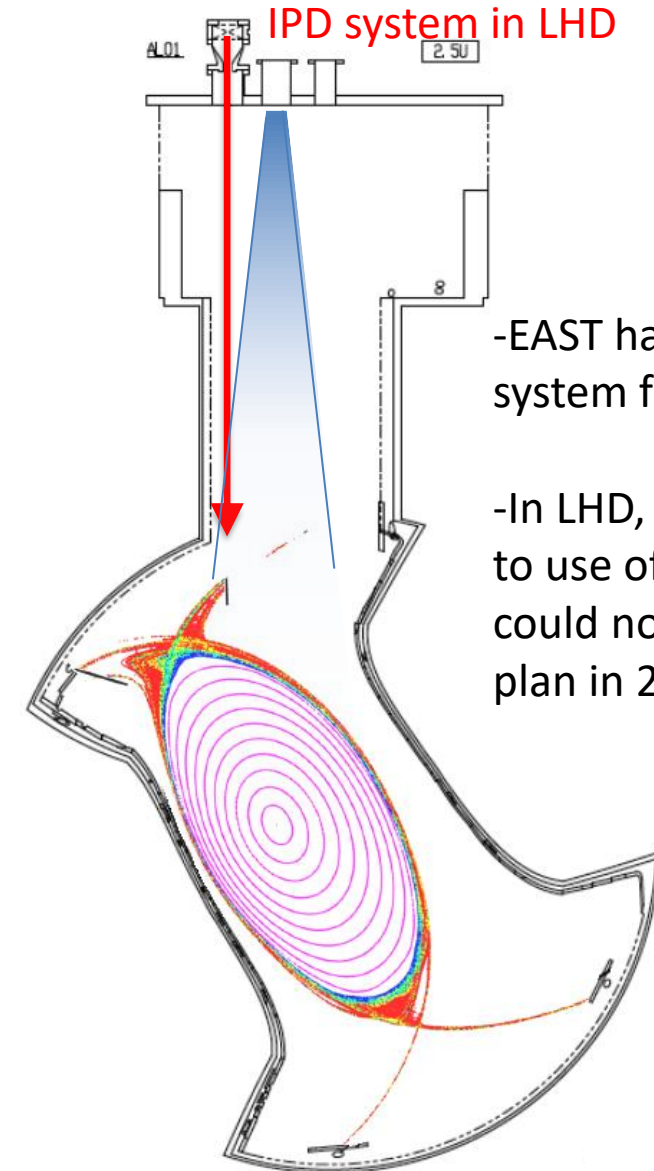
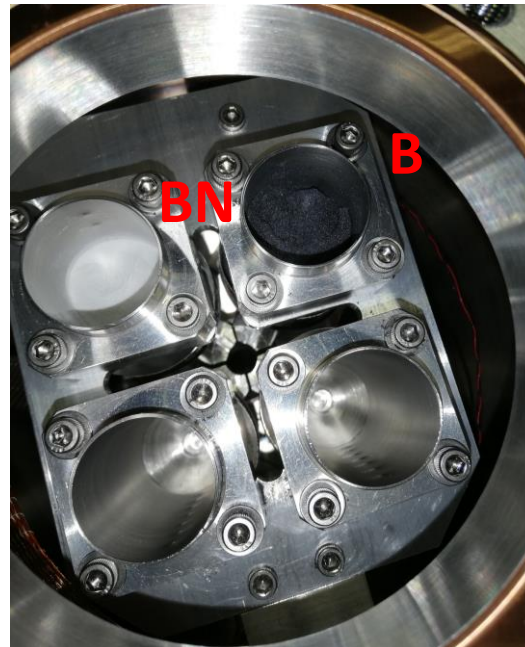
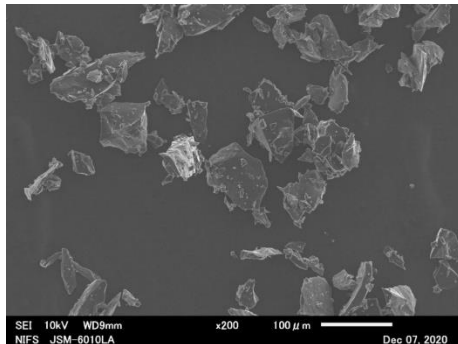
IPD was developed in PPPL and IPD-LHD experiments were started in 2019.

Powders are served to plasmas by the gravity force. A distance from the cup of powders to plasmas is more than 3 m

Dropped amounts of powders were controlled by the piezo vibration. At present, two materials, B and BN can be dropped in LHD

- 2~ 50 g boron (B) $\phi = \sim 100 \mu\text{m}$

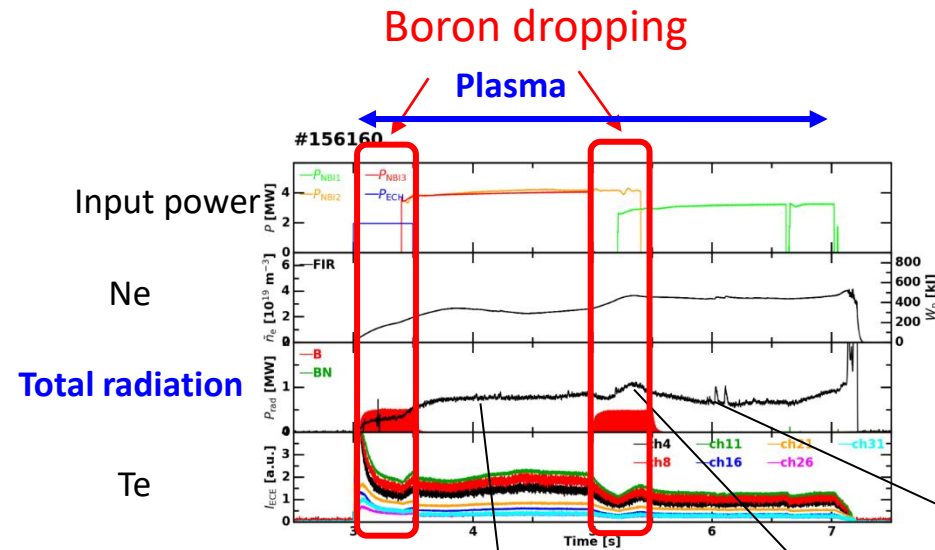
Boron powder



-EAST has the similar system for Li

-In LHD, we planed to use of Li, but could not use it. It plan in 2022.

IPD : Visible camera observation



Visible camera from 2.5 Upper port



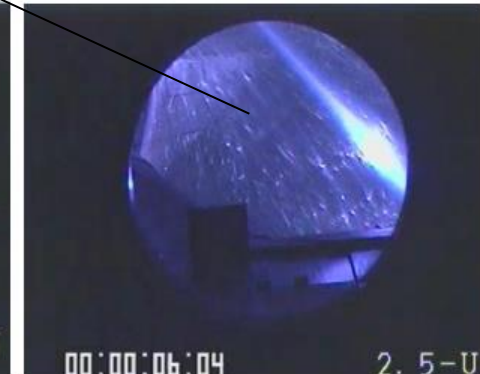
3:44s



4:04s



5:30s

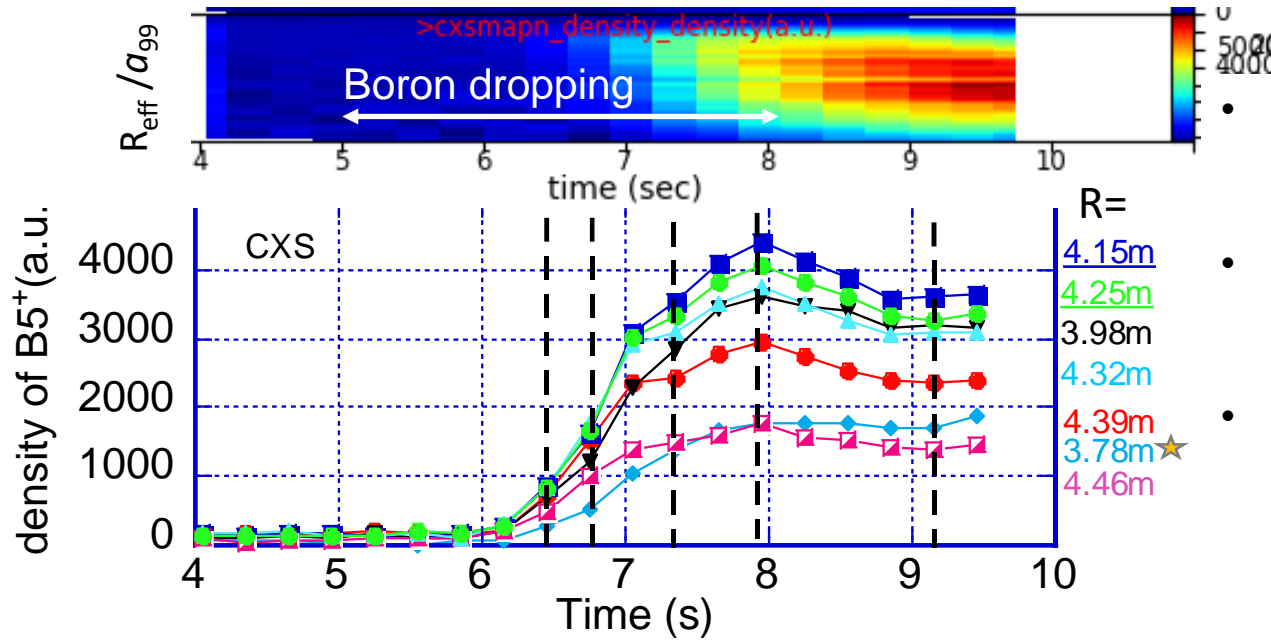


6:04s

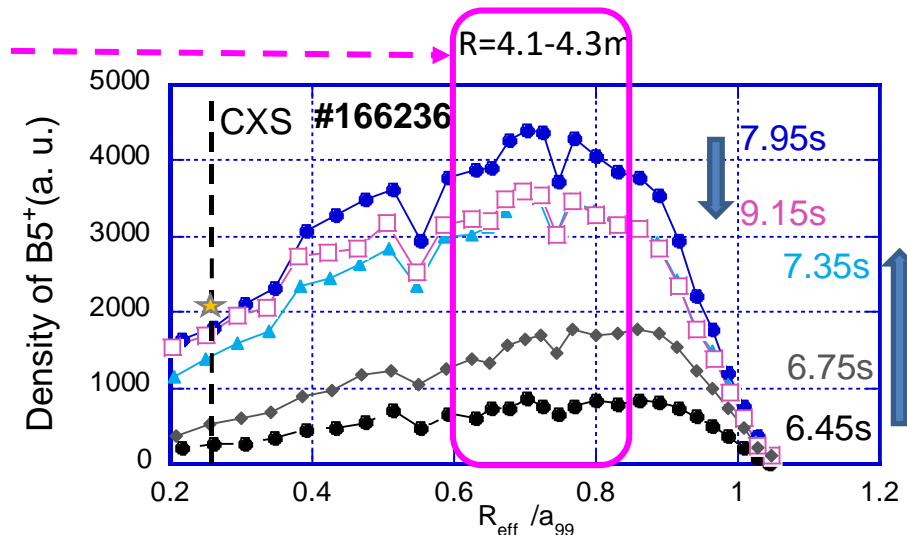
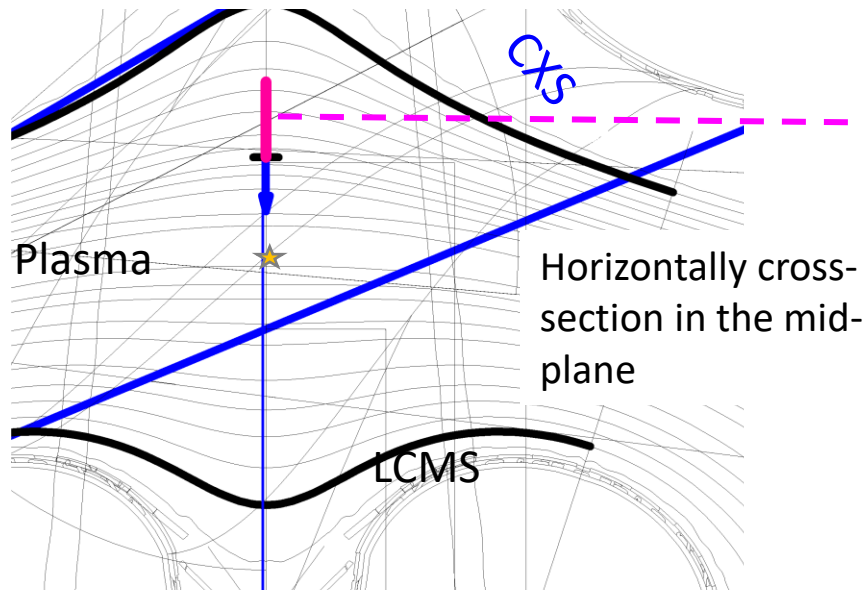
Starting time of Piezo vibration from 3s.

Starting time of brightness due to dropped powders about 4s.

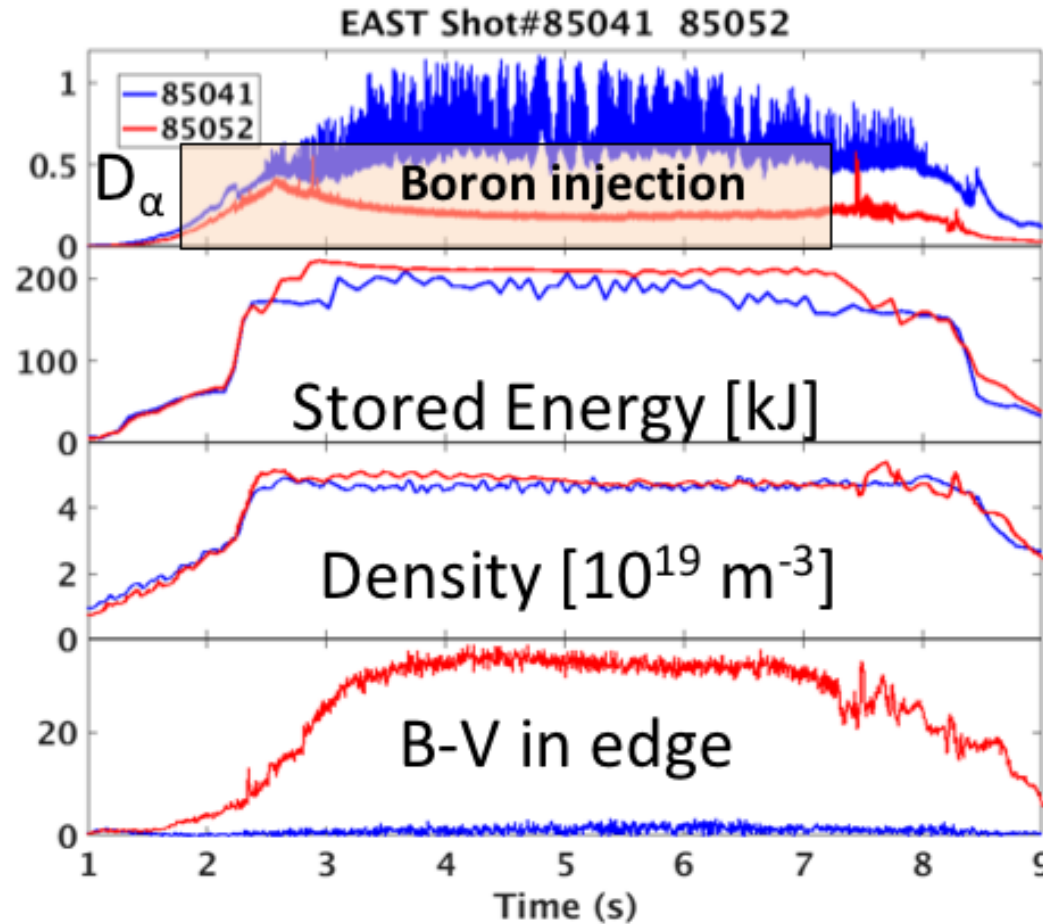
Spatial profiles of boron intensities (IPD)



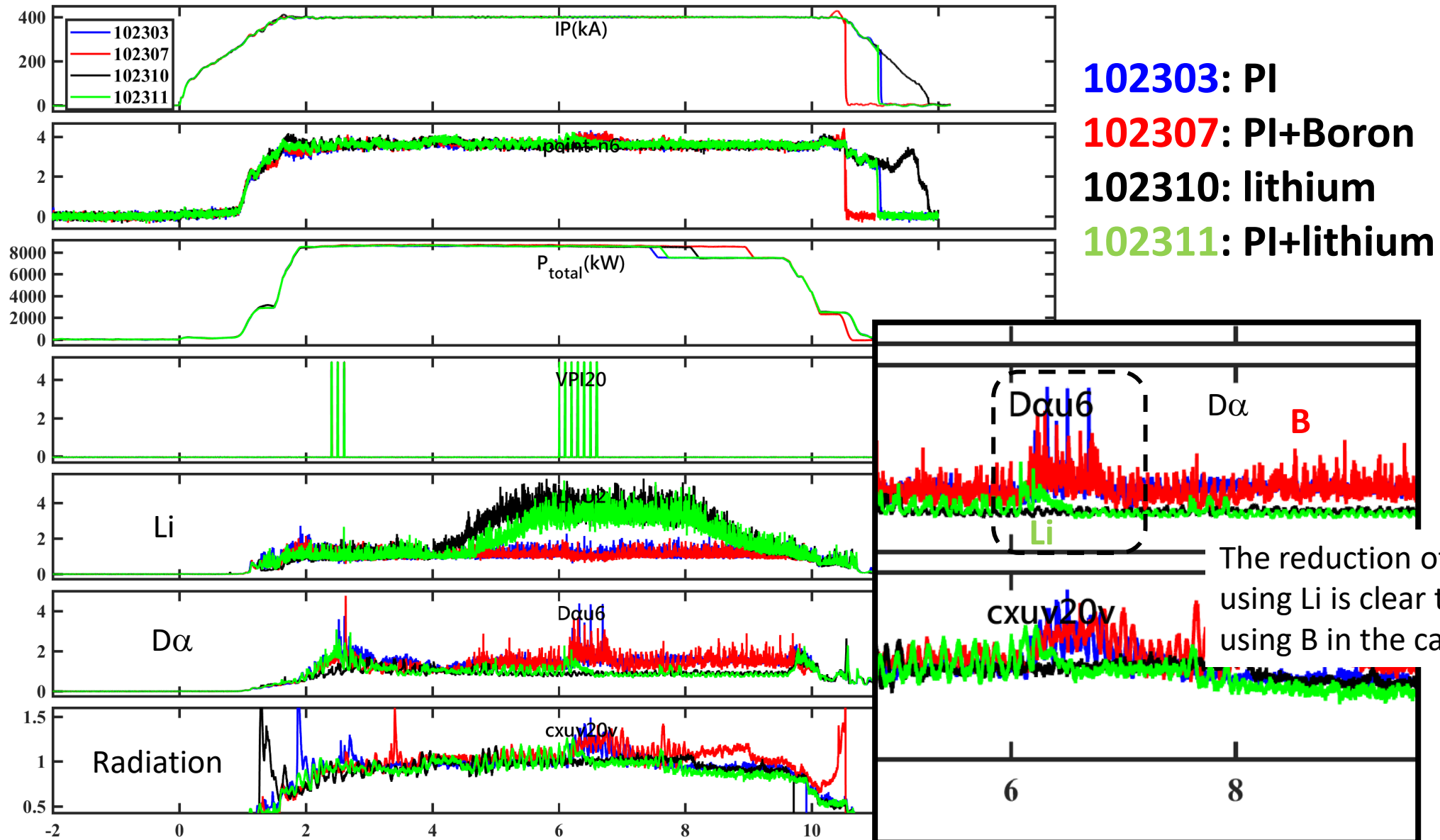
- From CXS observations, ionized boron is observed in the core plasmas.
- In $R_{\text{eff}}/a_{99} = 0.6-0.8$, density was increasing during B powder dropping
- In core region less than $R_{\text{eff}}/a_{99} = 0.4$, time evaluation is slower to compare with that in the outer region. The density level was kept after stopped the B dropping.



Boron dropping in EAST

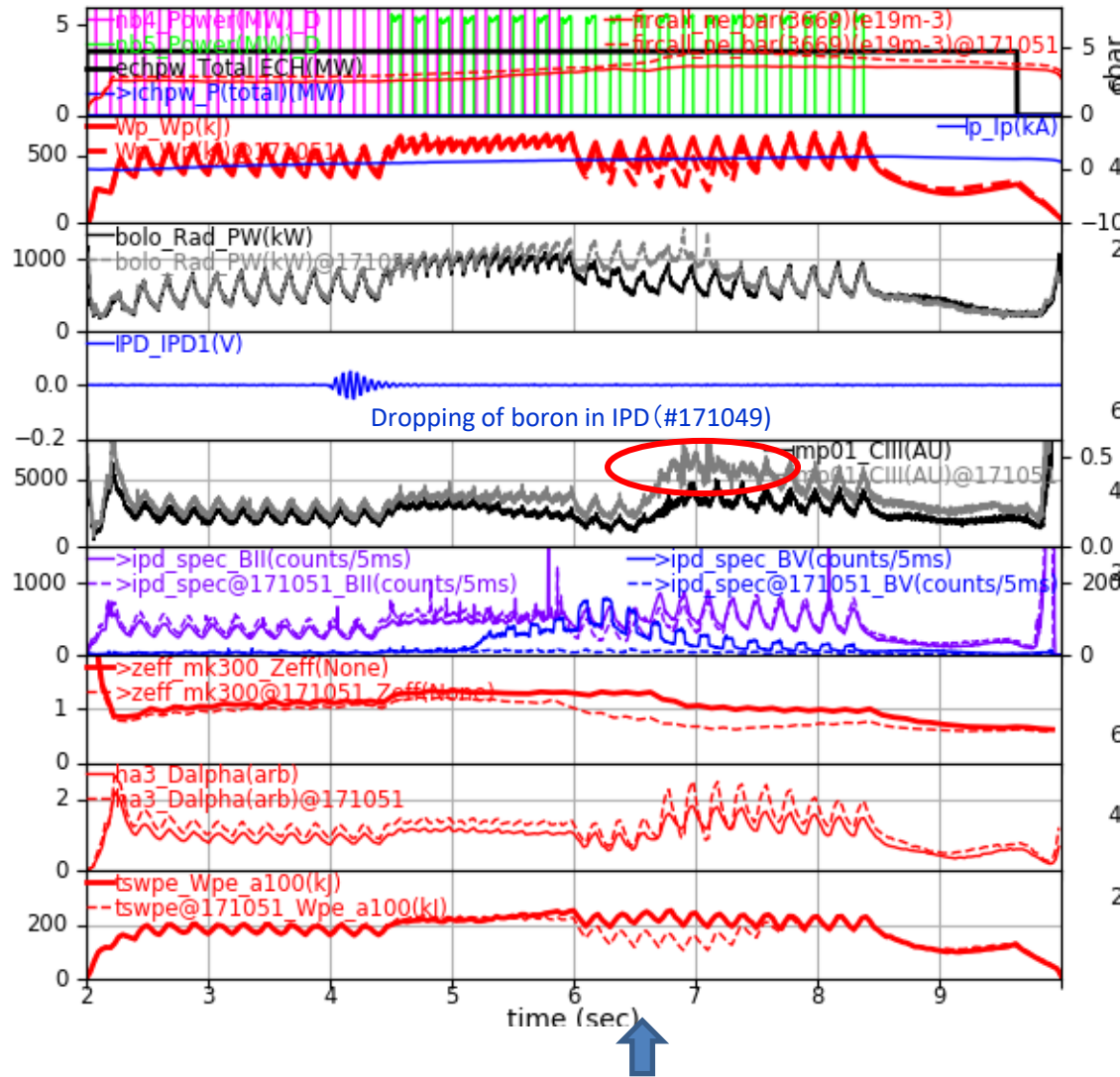


- No Boron (#85041)
- Boron injection above a threshold
- Stored energy increased slightly
- Density matched
- B-V in edge shows when B injected



LHD 171050 (Bt, Rax, gamma, Bq) = (2.75, 3.6,

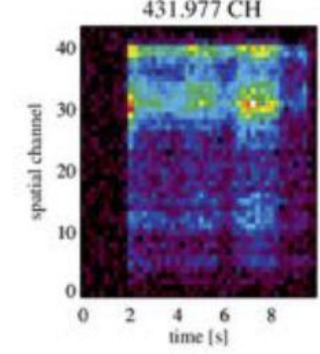
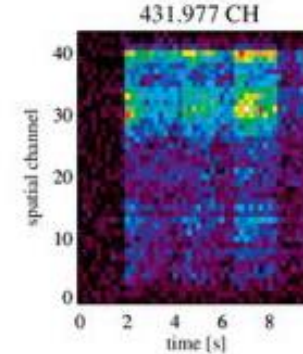
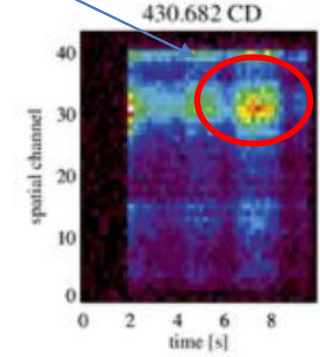
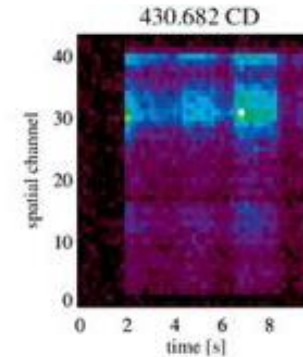
Increasing of CIII in the case of w/o boron at 7s.



Supersonic Gas puff (SSGP) for both #171050 and #171051 at 6.5s. After SSGP, Da increases at w/o B. How about He discharge? <<In the request sheet, He SSGP is written. If coating effect on the target is reason for a reduction of Da, it is might be useful for He.

#171050 w/

#171051 w/o



Increasing of CIII in the case of w/o boron
Kawate try to measure BH and CH lines

Progress on analytical method; Ti of B using BII data

T. Oishi, et al.,

Plasma Sci. Technol. 23 (2021) 084002

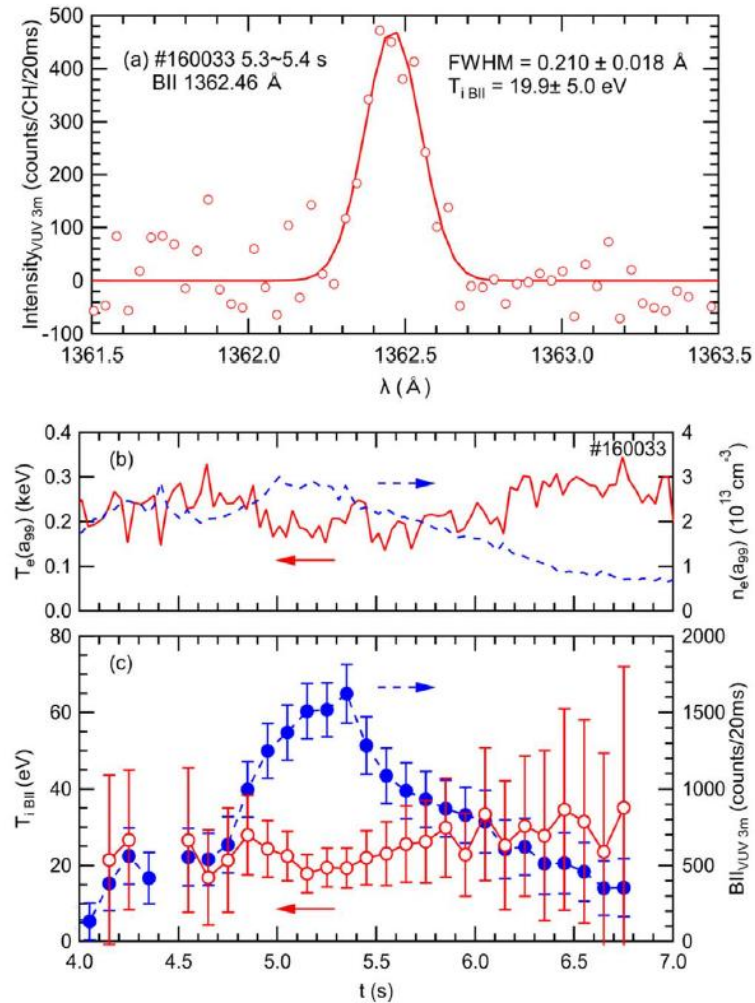
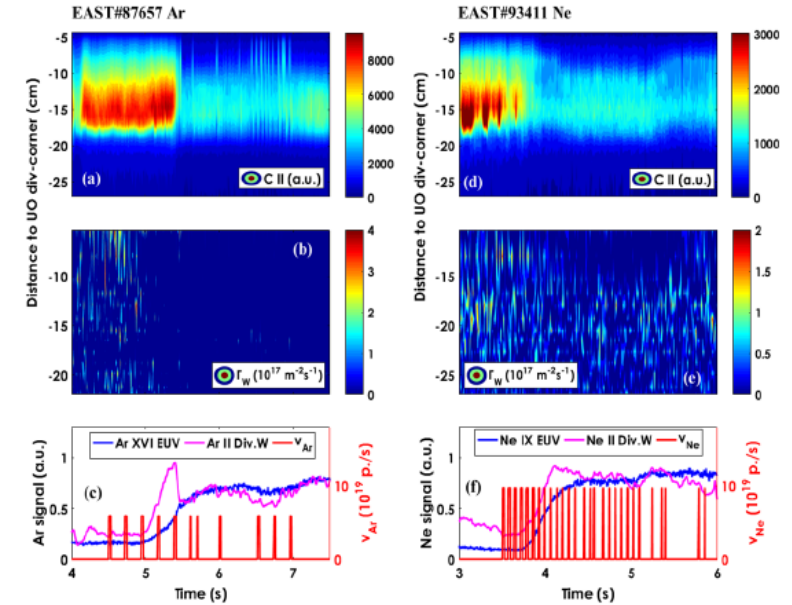
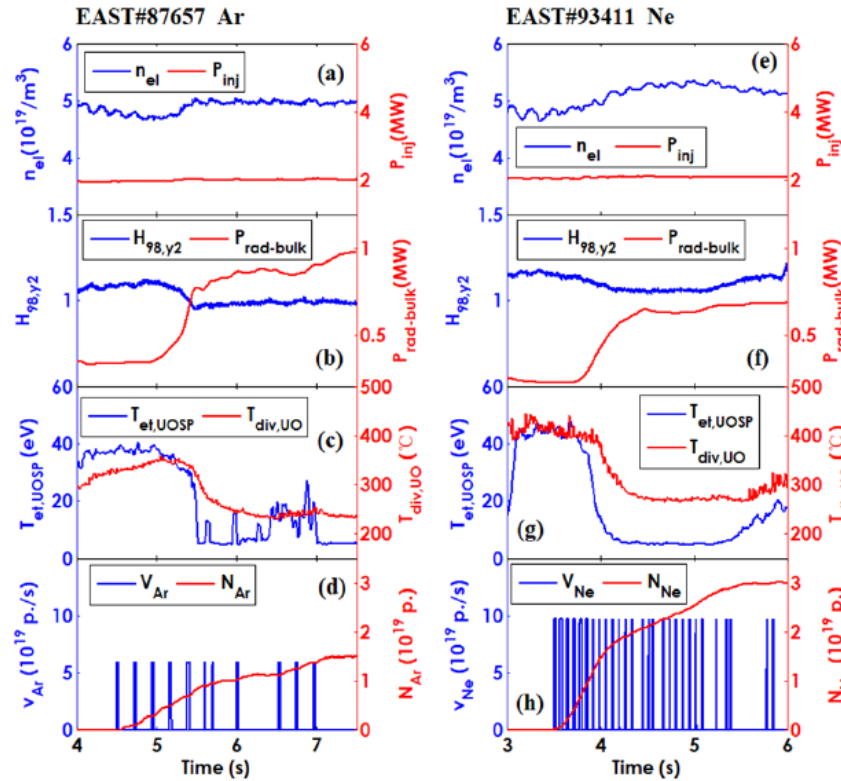


Figure 9. Ion temperature measurement of B^+ ions using a 3 m normal incidence VUV spectrometer. (a) Wavelength spectrum of BII 1362.46 Å together with a Gaussian function as a fitting curve. Temporal evolutions of (b) edge electron temperature $T_e(a_{99})$ and edge electron density $n_e(a_{99})$, and (c) the temperature of B^+ ions, T_{iBII} , and the emission intensity of the BII line. (d) Edge electron

The ion temperature T_i (eV) is provided by

$$T_i = 1.68 \times 10^8 M (\Delta FWHM / \lambda_0)^2,$$
 M is the atomic mass number,
 $\Delta FWHM$ is the Doppler width at FWHM,
 λ_0 is the central wavelength.

- ◆ $I_p=400\text{kA}$, $P_{inj}=2\text{MW}$,
 $B_t=2.5\text{T}$ (Fav.)
- ◆ $n_{el}\sim 4\times 10^{19}\text{m}^{-3}$
- ◆ $H_{98,y2}\sim 1$
- ◆ 50%Ar&50%D₂,
50%Ne&50%D₂
- ◆ $T_{et,UOSP}\sim 5\text{eV}$, Peak $T_{div,UO}\downarrow$



For energy detachment:

- Total number of impurity particles required:

$$N_{Ar} < N_{Ne}$$

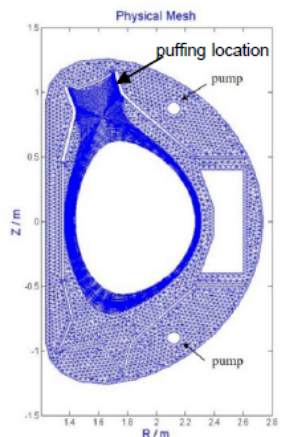
- Main plasma radiation increments:

$$\Delta P_{rad-bulk}(Ar) > \Delta P_{rad-bulk}(Ne)$$

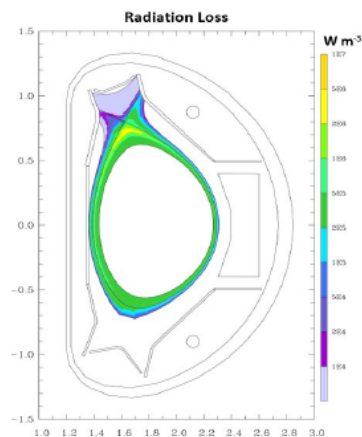
- Under current EAST operating conditions, better cooling capacity of Ar to the upper outer divertor and better compatibility between Ne and core plasma

- ◆ Ar, Ne radiation \uparrow by spec. data
- ◆ C radiation \downarrow
- ◆ $\Gamma_w: \sim 10^{17}\text{m}^{-2}\text{s}^{-1}$

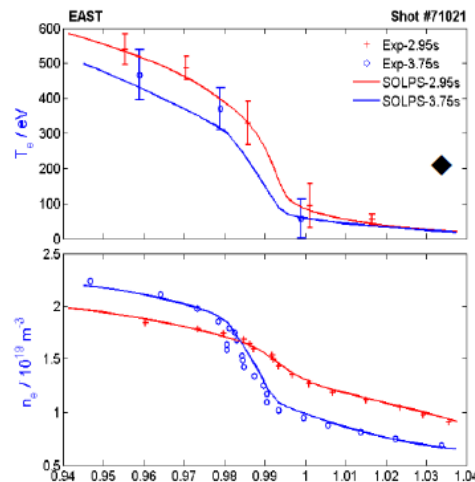
- Both Ar and Ne injection effectively suppress the sputtering of the divertor target plate



Computational mesh for Ne seeding for EAST #71021 ($t=2.95s$)

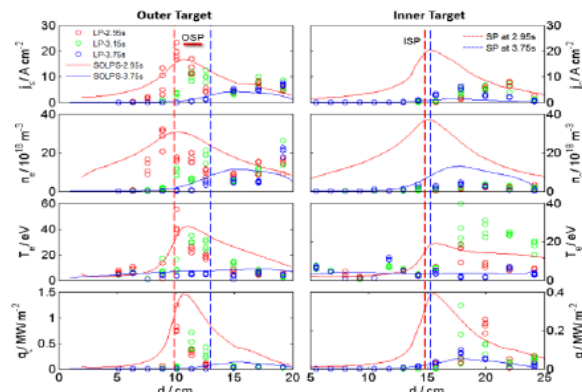


The modeled radiation distribution after Ne seeding (EAST #71021)



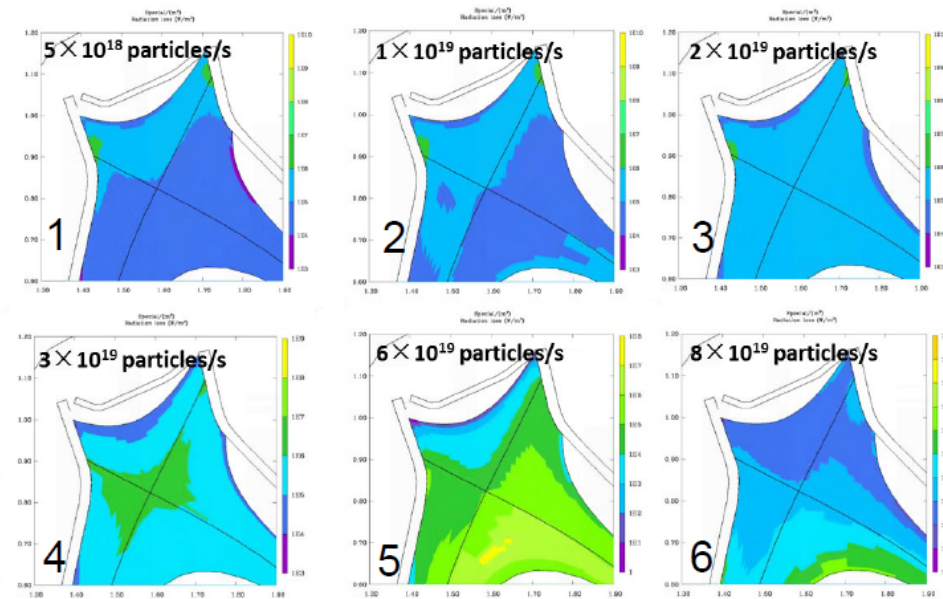
The upstream density and temperature

◆ The agreement between simulated midplane profiles and experimental results



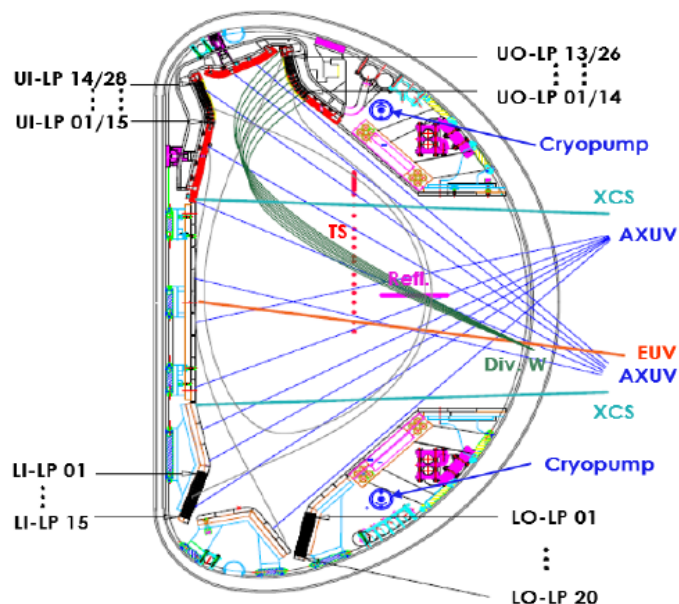
The density and temperature for outer /inner upper divertor target (experiments data vs. Simulation results)

Radiation distribution at different seeding rate



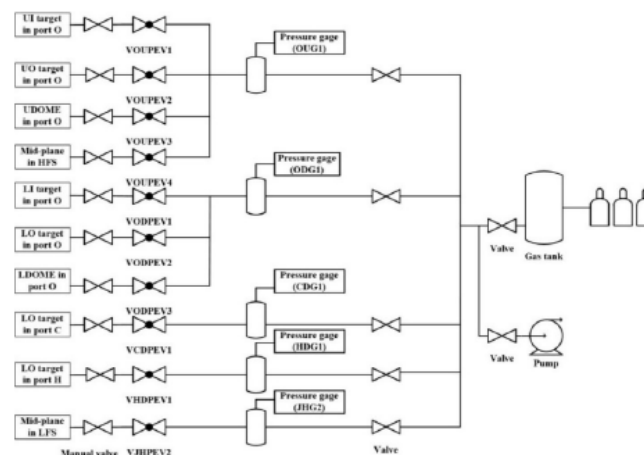
- ◆ **Lower frad*** (<40%): radiation mainly distributed in the divertor region (Fig.1-3) *frad=Prad/Ptotal
- ◆ **Medium frad*** (40%-65%): radiation mainly concentrated near the X point, and the MARFE phenomenon (Fig.4)
- ◆ **Higher frad*** (>65%) : Most of the radiation loss in the core region (Fig.5-6)

Impurity injection system and relevant diagnostics on EAST



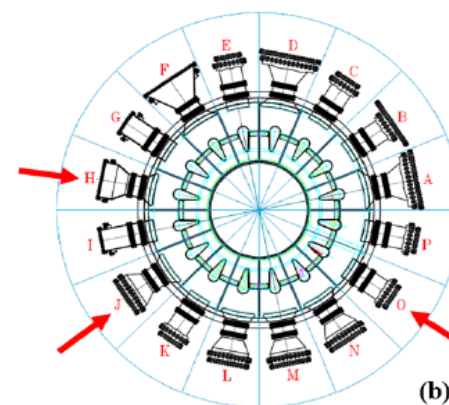
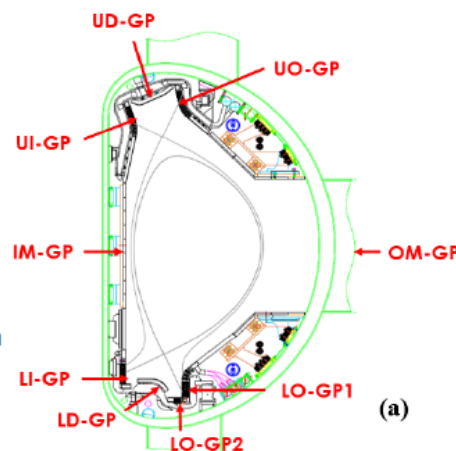
Diagnostics distribution for impurity injection experiments

- ◆ Divertor probe: j_z , T_e , n_e , q_t , p_{total} on the divertor target
- ◆ InfraRed camera: T_{div} , q_t on the divertor target
- ◆ Divertor tungsten spectrum: C, Ne, Ar, W in divertor region
- ◆ Extreme UltraViolet spectrometer, EUV: C, Ne, Ar, W in midplane
- ◆ Absolute eXtreme UltraViolet bolometer arrays, AXUV: radiation distribution
- ◆ Microwave reflectometer: n_e in midplane
- ◆ Thomson Scattering, TS: T_e in the core and boundary
- ◆ X-ray crystal spectroscopy, XCS: Ti in the core
- ◆ POINT: n_e in the core



- ◆ Impurity gas puffing system consists of: pipe, shut-off valve, piezoelectric valve, pressure stabilization tank, pressure gage and pump.

Impurity gas puffing system on EAST



- ◆ Toroidal: Port C, O, H
- ◆ Poloidal: UO, UI, UD, LO, LI, LD, IM

Gas puffing valves distribution in poloidal (left) and toroidal (right) directions on EAST

Requests from IAEA-2

- Which **database** services does the community use, need and want?
- Is the interaction of impurity species and/or their derivatives with reactor component surfaces in scope, such as formation and sticking of NH_x ; vapour formation and interaction with plasma species; etc ?
- How much experimental data is available, in what form, and can it (how can it) be **stored** and curated for **future use or analysis**?
- EAST ; on going
- LHD ; until FY2022 campaign (it is not clear after FY2023 due to the budget. But Stored data is the data repository