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Hydrogen retention in RAFMs – Current status of investigation



Fusion investigations (tokamaks) give no visible benefit for humanity for decades

Nobody can promise that fusion power plant will be built and work properly within 20, 50 years

The only direction where fusion can give practical use relatively quickly, in our lifetime – hybrid reactors with a tokamak as a neutron source for fission (subcritical nuclear reaction & radioactive waste processing)



Status of DEMO-FNS Development

Head of the project: B.Kuteev

Fusion-fission hybrid facility based on superconducting tokamak DEMO-FNS is under development in Russia fusion power up to 40 MW fission power up to 400 MW



FEC-26 Yu.S. Shpanskiy et al. FIP/O7-4

Aspect ratio R/a, m	3.2/1
Toroidal magnetic fi	eld 5 T
Electron/ion	
Temperature, keV	11.5/10.7
Beta normalized β_{N}	2.1
Plasma current I _{pl} ,	5 MA
Neutron yield G _N	1.3·10 ¹⁹ /s
Neutral injection po	wer 36 MW
ECR heating power	6 MW
Discharge time	5000 h Capacity
factor 0.3	
Life time	30 years
Consumed/	
generated power	200 MW

Facility is considered in RF as main source of technological and nuclear science information that complement the ITER research results in the fields of burning plasma physics and control



ITER, FNS and DEMO in neutron damage

	ITER	DEMO-FNS	DEMO	FBR
Fusion power	0.5 GW	0.05 GW	2-5 GW	-
Heat flux (first wall/divertor)	0.2 /10 MW/m²	0.2 /10 MW/m ²	0.5 /20 MW/m²	-
Neutron load (First wall)	0.8 MW/m²	0.2 MW/m ²	2 MW/m ²	
Integrated Neutron load (First wall)	0.07 MW·y/m ² (3-years operation)	2 MW∙y/m²	5-8 MW∙y/m²	
dpa	4 dpa	∼ <mark>30 dpa</mark> (3-3.2 dpa/fpy)	50-80 dpa	80 dpa/fpy
He production	40 appm	300-400 appm	500-800 appm	3 dpa/fpy
	[D.Stork]	[Yu. Spanskiy, V. Khripunov]	[D.Stork]	[V. Chernov]



ITER, FNS and DEMO in neutron damage

Current status: from pre-conceptual to technical design (different systems)

Main efforts: to provide maximum neutron flux

Current status of FNS First wall:



Problem: properties of bronze properties become unacceptable after a few d.p.a.

A use of steel instead of bronze is discussed

Problem: thickness of steel is limited to provide necessary thermoconductivity, by 1 mm

RAFMs can be used without Be cladding in some remote areas



Damage created at work of a fusion device

(in particular at neutron irradiation, radiogenic He formation in materials)

will change parameters of H-material interaction

The aim of the project: Modelling of influence of neutron damages on deuterium retention in RAFMs for fusion

1. Radiogenic He arise

- 2. n-irradiation (by Fe and H ions irradiation)
- **3. Selective sputtering study**



Helium will appear in the bulk of fusion materials as a result of tritium decay

$$_{1}T^{3} \rightarrow _{2}He^{3} + e^{-} + v$$

The best way to introduce He is "tritium trick" (introduce tritium, wait for He formation, remove remaining T at annealing)

Main advantages:

- No additional defects (as it would be at He implantation
- He is distributed over all material (at ion or plasma irradiation it would be only in a thin layer)



Installation for exposure in tritium gas



A.A. Bochvars institute of Inorganic Materials, Russia



Installation is designed and mounted in a special room with a reduced pressure (to prevent T leakages outside)

- Container with T cooled in liquid nitrogen 1.
- 2. Container is filled with T gas (5 atm)
- Container if heated up to necessary T (P^{\uparrow}) 3.

Heater; inside the container with samples, filled with T gas

inert gas



Materials for investigation:

Rusfer (deployed tube) Eurofer





New portion of Rusfer was produced recently. Flat rolled material can be available

BUT

There are at least 10 standard procedures of thermal treatment, leading to different thermomechanical properties.

What are the requirement to RAFMs properties as a material of next-step fusion device?



Test exposure in D gas



- Repeatability of results
- Retention:

Bronze>Austenitic steel>RAFMs>>W



Test exposure in D gas



Do define weather we reach a full saturation:

ChS-68, Rusfer, W, CuCrZr P = 20 atmospheres T = 673 K (400 C) t = 100 hours



TDS, MEPhI



Test exposure in D gas



- Total retention is slightly higher for 100 hour exposition but is within the error for all materials.
- For rational use of radioactive tritium and of time there is no science to expose samples in T (20 atmospheres, 673 K) longer than 24 hours



Conditions: T= 673 K (400 C), P= 8 atmospheres, 24 hours

Exposed samples: Eurofer (Dr. Mayer, IPP) 10x10x1 mm Rusfer (Ek-181) (some of them pre-damaged by 10 MeV electrons) 10x10x0.5 mm (deployed pieces of tubes) Austenite ChS-68 (some of them pre-damaged by 10 MeV electrons) 10x10x0.5 mm (deployed pieces of tubes) W (Goodfellow) 10x10x1 mm **Bronze (ITER grade)** 10x10x1 mm



Conditions: T= 673 K (400 C), P= 8 atm, 24 hours

How to define initial amount of tritium in samples?

$$_{\rm L}{\rm T}^3 \rightarrow {}_{\rm 2}{\rm He}^3 + {\rm e}^- + {\rm v}$$

 $E_e = 5.7 \text{ keV}$

- Geiger counter can't detect electron of so small energy
- In Bochvar's institute (though they have many detectors of βparticles) - no available detectors for such a small energies
- Estimation?

Sieverts low: $C=S\cdot p^{1/2}$

- No data for solubility at high pressures for RAFMs for fusion
- Problematic to extrapolate (8 orders of magnitude higher pressure)





Conditions: T= 673 K (400 C), P= 8 atm, 24 hours

Method to measure activity: β-induced radioluminescence

using imaging plate (IP)



Normally

P ~ tens of Pa, hours exposure in T, hours of exposure samples to IP for

gives quantitative results using calibrative pattern



Conditions: T= 673 K (400 C), P= 8 atmospheres, 24 hours

T amount at surface is really huge, a minute is enough to light IP Samples was just pressed to IP for 10 seconds – very big uncertainty of measurement



Rusfer #3

ChS-68 #3

Eurofer



What we can do:

- 1. Make a quality profiling at a cross-section using IP
- 2. Wait for formation of radiogenic He (year)
- 3. Make He depth-profiling using NRA (few μm layer below surface)
- 4. Define initial depth-profile of T (when we know time of T decay and final He concentration)
- 5. The shape of depth-profile of He should be the same as of T \rightarrow restore profile across the sample



A very time-demanding experiment (T $_{\rm 1/2}$ ~12 years)

- 1. Exposition in T at elevated pressure and selected temperature
- 2. Image of surface and a cross-section of samples
- 3. Extracting a sample of each type every 0.5 year, detritization
- 4. If it is proven that the residual activity meets the security requirements
 - 4.1. Depth-profiling of He
 - 4.2. Loading with D gas, irradiation with D plasma
 - 4.3. Compare retention in samples containing radiogenic He and as received

Now we are waiting for first half-year of He production from tritium





the Institute for Theoretical and Experimental Physics (ITEP), T. Kulevoy, P. Fedin belongs to Kurchatov institute

Fe²⁺ accelerator

heavy ion RFQ HIP-1 (Radio Frequency Quadrupole Heavy Ion Prototype) with metal ion beams generated by ITEP Metal Vapor Vacuum Arc (MEVVA)

M/Z ≤ 60 E = 101 keV/a.m.u. E(Fe²⁺) ≤ 5.6 MeV E(W ions) = 19 MeV Ion beam density 3–6 mA/cm² Ion beam pulse length is 450 μ s 1 pulse for 4 sec P ≤ 2 ×10⁻⁶ mbar







T. Kulevoy, P. Fedin

1 sample

1 sample $10 \times 10 \text{ mm}$ (current on the sample is $80 \ \mu\text{A}$) $\rightarrow 1$ hour for fluence 10^{14}cm^{-2} . The current density changes by 2.5% over a sample.

4 samples

4 sample 10×10 mm (current on the sample is $120 \ \mu A$) \rightarrow 3 hour for fluence 10^{14} cm⁻². The current density changes by 11% over a sample



current density distribution over a sample





8

10

85.0%

80.0%

0

2





5x10⁶

0

2 mm

	H+ linear	accelerator I-2	T. Kulevoy, P. Fedin		
E, MeV	0.7÷24.6				
I, mA	200				
t, мкс	10				
Ω, Hz	<1				
I, particle/s	10 ¹³	SRIM modelling			
Length of resonator, m	18.4	Depth distribution of damage by			
Diameter of resonator, m	1.37	24,5 MeV H of SS (Fe-12%Cr-1%W)			
	FEIL	COLLISIO	NEVENTS		

 $-I \circ n$)

Number/(Angstrom)

d V

0 A



Depth distribution of damage by 24,5 MeV H of SS (Fe-12%Cr-1%W) COLLISION EVENTS Vacancies Produced (K-P) 30x10⁻⁶ 25x10⁻⁶ 20x10⁻⁶ 15x10⁻⁶ 10x10⁻⁶

- Target Depth -



- 1. Investigation of influence of radiogenic He of D retention in RAFMs at exposure in gas and plasma irradiation
- 2. Investigation of influence of damaging by MeV H and Fe ions on D retention in RAFMS





LENTA, Kurchatov insitute B.I. Khripunov

Iradiations of Rusfer samples:

E=50 eV Plasma – H or D Flux - 2,5 \cdot 10 ²¹ at./m²s Fluence - 7 \cdot 10 ²⁴ at./m² T₁= 320 K, T₂= 600 K











D-plasma irradiation, 600 K, 5·10²¹ D/s·m², 7·10²⁴ D/m², LENTA facility Kurchatov institute

Irradiated sample after TDS was coated with Pt, cut with focused ion beam (FIB)



Specification: Rusfer contains 1.13 wt.% of W

Energy dispersive x-ray spectroscopy (EDX):

5 nm layer below surface

Element	Weight %	Atomic %
Cr(K) Fe(K) W(M)	8.52 89.83 1.64	9.20 90.29 0.50



D-plasma irradiation, 600 K, 5·10²¹ D/s·m², 7·10²⁴ D/m², LENTA facility

Kurchatov institute

Irradiated sample after TDS was coated with Pt, cut with focused ion beam (FIB)



 Layer 50-150 nm below surface

 EDX:
 EDX:

 Element
 Weight %
 Atomic %

 Cr(K)
 10.22
 11.07

 Fe(K)
 87.46
 88.21

 W(M)
 2.31
 0.70

Probably, partially Pt-area was included in averaging

Uncertanty, even with a reference samples for normalization – 1 at.%



D-plasma irradiation, 600 K, 5·10²¹ D/s·m², 7·10²⁴ D/m², LENTA facility

Kurchatov institute

Irradiated sample after TDS was coated with Pt, cut with focused ion beam (FIB)





- 1. Investigation of influence of radiogenic He of D retention in RAFMs at exposure in gas and plasma irradiation
- 2. Investigation of influence of damaging by MeV H and Fe ions on D retention in RAFMS
- **3.** Try to find possibility define surface concentration of W



Thank you for your attention!



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