

Deuterium retention in tungsten damaged by high-energetic tungsten ions

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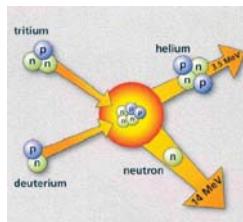
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Neutron damage of tungsten



Irradiation of tungsten by fast neutrons results in

- Displacement damage
⇒ Vacancies, vacancy clusters, interstitials, ...
ITER: Maximum ~ 1 dpa
DEMO: ≤ 15 dpa per 1 fpy M.R. Gilbert JNM (in press)
- (n,α) reactions
⇒ Production of helium in the bulk of the material
DEMO: ≤ 4 ppm He per 1 fpy M.R. Gilbert JNM (in press)
- Transmutation products
⇒ Rhenium, Osmium

Displacement damage by heavy ions often used as proxy for neutrons

B. Tyburska JNM 395 (2009) 150; O.V. Ogorodnikova JNM 415 (2011) S661; V.Kh. Alimov JNM 420 (2012) 370; ...

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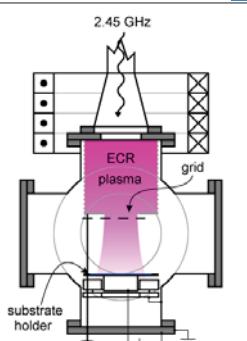
Experimental: Deuterium implantation

Low-temperature ECR plasma (PlaQ):

- ion flux: $0.5 - 1 \times 10^{20} D/(m^2 s)$
97% as D^{3+}
2% as D^{2+}
1% as D^+

A. Manhard, Plasma Sources Sci. Technol. **20** (2011) 015010

- atom flux $> 10^{21} D^0/(m^2 s)$
- energy: „ $< 5eV/D$ “ (floating)
- $T = 400-450 K$ (liquid thermostats)



“Gentle” implantation of deuterium

⇒ Minimal additional damage by implantation

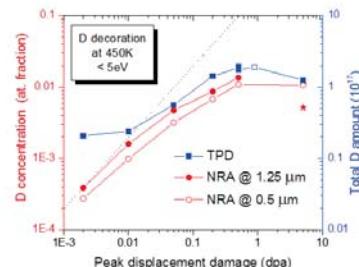
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Retention in damaged W

- Retention in displacement damage saturates at 0.5 - 1 dpa

T. Schwarz-Selinger, PFMC 2013



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Motivation: Tritium Inventory in ITER



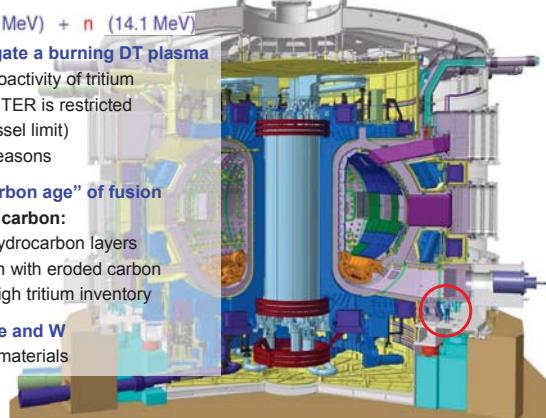
- ITER will investigate a burning DT plasma

⇒ Problem: Radioactivity of tritium
⇒ T-inventory in ITER is restricted to 700 g (in vessel limit)
due to safety reasons

- 1990 – 2010: “Carbon age” of fusion

⇒ Knock-out for carbon:
Formation of hydrocarbon layers by codeposition with eroded carbon results in too high tritium inventory

- ⇒ ITER will use Be and W as plasma-facing materials



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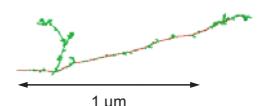
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Simulation of Neutron effects by Ions

Creation of a collision cascade, starting with primary knock-on (PKA)

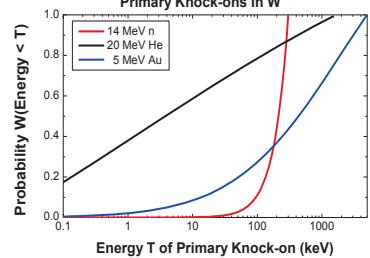
- PKA spectrum has some similarities between n and W
- ⇒ Kinetic aspect can be simulated (to some extend) with ions

20 MeV W in W



Differences between n and ions:

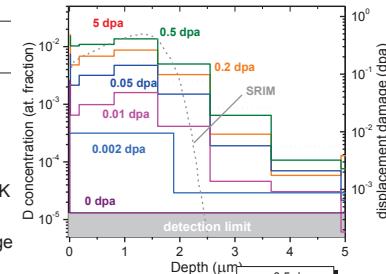
- He-effects, transmutations cannot be simulated directly by ions
- Ions deposit energy by electronic energy loss, ~9 keV/nm @ 20 MeV
⇒ local heating around ion track



S. Bouffard, private communication

Retention in damaged W

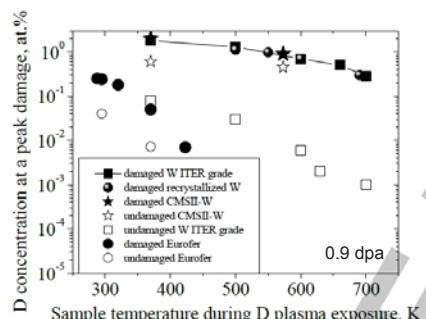
- Hydrogen retention in natural W: 0.001 – 0.01% at $\leq 450 K$
- Displacement damage increases hydrogen retention to 1.5% at 450 K
⇒ In the presence of neutrons retention is dominated by damage



T. Schwarz-Selinger, PFMC 2013

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Retention in damaged W (2)



- Retention at ~1 dpa in displacement damage is independent of initial material

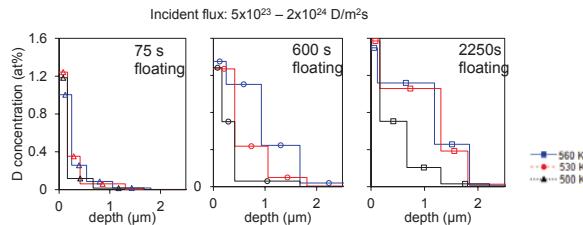
O.V. Ogorodnikova, JNM, in press

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High flux effects

IPP



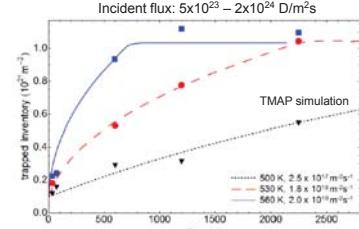
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High flux effects (2)

IPP

- At high fluxes: Slow diffusion inwards
- Only very small fraction $\sim 10^{-5}$ of incident D enters the target
⇒ Surface energy barrier
M.H.J. 't Hoen, in print at PRL
- Very high fluences necessary for saturation of damaged layer

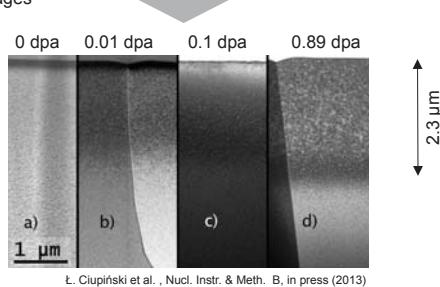


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Self-damaged tungsten: Dislocations

IPP

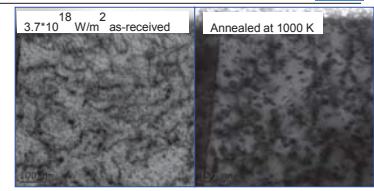
STEM bright field images



Self-damaged tungsten: Annealing of defects

IPP

- Annealing of defects not significant below 800 K
- 30% remaining H-decorable damage after annealing at 1200 K
- Residual damage still visible in TEM after annealing at 1000 K
⇒ dislocation loops, long dislocations



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Evolution of microstructure: coarsening up to damage levels of 6 dpa

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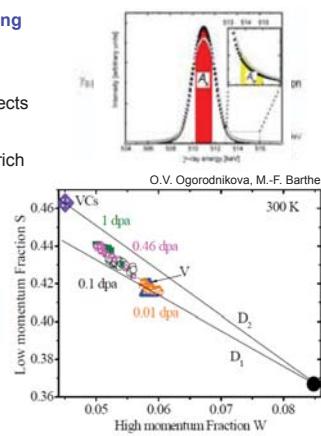
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Investigation of defects: Positron Annihilation

IPP

• Positron Annihilation Doppler broadening spectroscopy

- Core electrons:**
high momentum, broad spectrum, low defects
- Valence electrons:**
low momentum, narrow spectrum, defect-rich
- 0.01 dpa: Mainly single vacancies, few vacancy clusters
- 1 dpa: Increase of vacancy clusters with $n < 30$
- No large clusters with $n > 30$

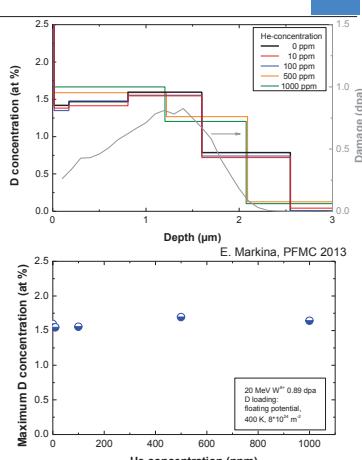


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Influence of helium on maximum concentration of D

IPP

- 0.89 dpa damage by 20 MeV W $^{6+}$
- Implantation of D until full saturation of all available traps
- Maximum D concentration $\sim 1.6\%$ at 400 K independent of He amount
⇒ Amount of additional trap sites by He is small compared to trap sites by dpa



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Influence of helium on D retention

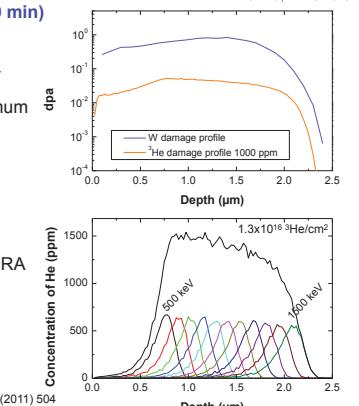
IPP

Recrystallized W samples (2000 K, 10 min)

E. Markina, PFMC 2013

Displacement damage by 20 MeV W $^{6+}$

- 1.4×10^{14} ions/cm $^2 \sim 0.9$ dpa in maximum



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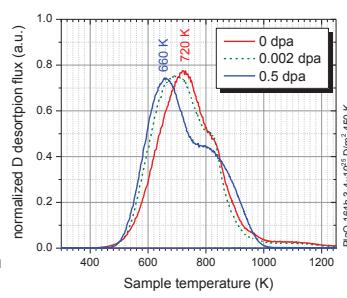
Problem: Determination of trap energies

IPP

Common practice in this community:

- Assigning trap binding energies to peak temperatures
- Assigning defect nature to trap energies
- Typically between 0.8 and 2.1 eV for damaged tungsten

Drawback: input parameter unknown (pre-exponential factor ν)

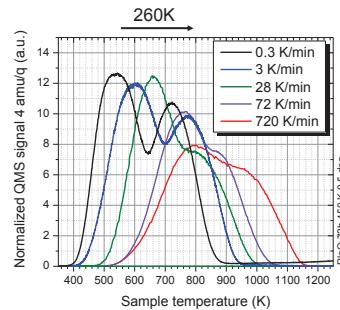


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"Real TPD": Measuring adsorption energies on surfaces

- Different T ramps
- Determine shift of peaks
- Derive trap energies from this peak shift
(Redhead or Falconer and Madix method)



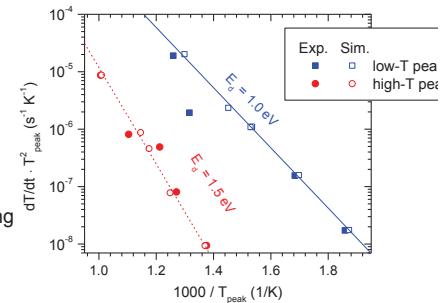
"Real TPD" analysis according to Falconer and Madix* reveals:

- Two trap energies

- $E_d = 1.0 \text{ eV}$
- $E_d = 1.5 \text{ eV}$

- Pre-exponential factors:
 $1 \times 10^{10} \text{ s}^{-1}$, $6 \times 10^{10} \text{ s}^{-1}$

- Diffusion trapping modeling shows applicability of the method

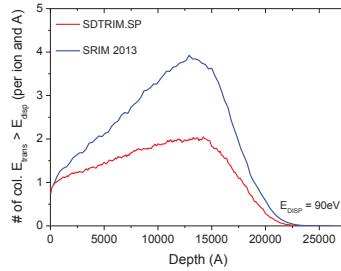


* J.L. Falconer and R.J. Madix, Surf. Science 48 (1975)

Problem: Damage profile

Concern: SRIM versus SDTrim.SP

- Two similar codes deliver different damage profiles although
- Both use Ziegler-Biersack stopping power
- ????



Summary

- Damage by 20 MeV W ions + gentle implantation of D at 450 K
- 1.5% D/W at ~1 dpa for 450 K
- Retention saturates at 0.5 – 1 dpa
- Retention at ~1 dpa is independent of initial material
- High flux effect: Surface barrier
- Evolution of microstructure: coarsening up to 6 dpa
- Annealing of defects not significant below 800 K
- 30% remaining H-decorable damage after annealing at 1200 K
- PAS: Increase of vacancy clusters with $n < 30$ at 1 dpa
- No additional influence of He up to 1000 ppm
- "Real" TPD using Falconer/Madix method yields different energies/pre-exponential factors than usually assumed
- SRIM 2013 and SDTrim.SP give different dpa: ???