



# Thermal desorption of deuterium from damaged tungsten: experiments and calculations

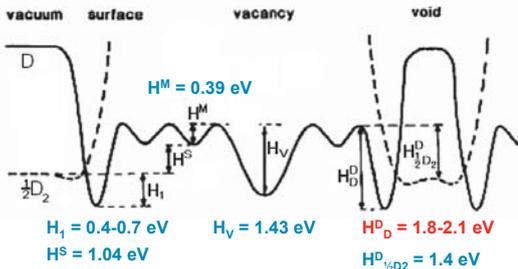
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### Outline:

- Introduction
- Damage by keV ion beam
- Damage by MeV ion beam (collaboration with IPP)
- Calculations (Diffusion equation)
- DFT calculations

## Deuterium retention in tungsten



A. Van Veen et al., JNM, 155-157 (1988) 1113-1117.

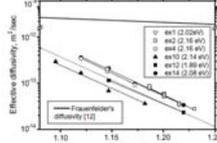
- Tungsten has a very low solubility ( $H_s = 1.04 \text{ eV}$ ), and rather strong traps!
- **Trapping determine accumulation of hydrogen isotopes in tungsten!**
- Traps: dislocations, grain boundary, vacancy, vacancy clusters, voids.
- One trap can accumulate more than one atom of deuterium.

## Filling of traps

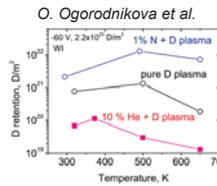


- Traps retard transport of hydrogen in the bulk of material.
- The effective diffusivity can be 1-2 orders less than real Frauenfelder's value.
- ➔ Even if we have a high concentration of traps in the bulk, the filling rate of these traps can be slow.

Yu.M.Gasparyan et al., JNM, 390-391(2009) 606-609.



- Surface conditions are important for the filling rate! Impurities in the incident flux can significantly change the filling rate of traps.
- Different impurities can both increase and decrease the filling rate and retention

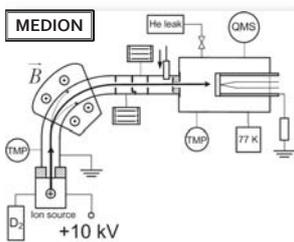


- Yu. Gasparyan et al, Journal of Applied Physics 110, 33303 (2011).
- O. V. Ogorodnikova et al., Phys. Scr. T145 (2011) 014034.

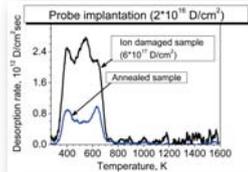
## Damage by keV $D_3^+$ ions: Probe method



A. Rusinov, Yu. Gasparyan, et al., JNM, 415 (2011) S645-S648.



- Ion beam:** 10 keV/ $D_3^+$ ,  $10^{14} \text{ D/cm}^2\text{sec}$ ,  $\varnothing 3\text{mm}$
- TDS:** 2 K/sec, 300K-1750K, ~10 min after irradiation
- Probe fluence:**  $2 \times 10^{16} \text{ D/cm}^2$
- Probe TDS** reflects types and concentrations of defects



## Participants of the project

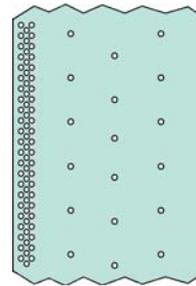


### MEPhI

- Plasma Physics department (Yu. Gasparyan, A. Pisarev, E. Marenkov, V. Efimov, A. Mednikov)
- Material Science department (M. Ganchenkova)

### IPP (O. Ogorodnikova, M. Mayer, K. Sugiyama)

## Ion and neutron damage in W



- **Ion induced defects** are located in the limited area close to the surface with the concentration up to several at. %
- **Neutron induced defects** are distributed deep in the bulk of material

$$100 \text{ m}^2 \times 1 \mu\text{m} \times 10\% \frac{n_{\text{trap}}}{n_{\text{solid}}} = 3 \text{ g}$$

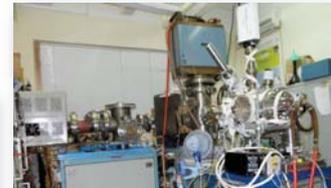
$$100 \text{ m}^2 \times 1 \text{ cm} \times 0.1\% \frac{n_{\text{trap}}}{n_{\text{solid}}} = 300 \text{ g}$$

- What kind of traps do neutrons produce? (Difference from ions?)
- Hydrogen transport in damaged W? (Filling rate of traps)
- Evolution of traps with the temperature?

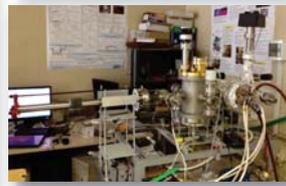
## TDS facilities in MEPhI



**UHV TDS stand**  
(TDS of externally exposed samples)



**MEDION**  
(keV ion implantation + *in situ* TDS)

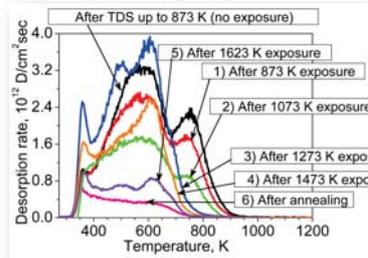


**MD-2**  
(Co-deposition of D with materials + *in situ* TDS)

## Damage by keV $D_3^+$ ions



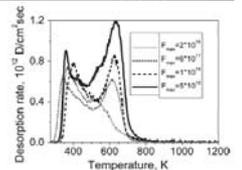
**Sample, damaged by  $6 \times 10^{17} \text{ D/cm}^2$**



A. Rusinov, Yu. Gasparyan, et al., JNM, 415 (2011) S645-S648

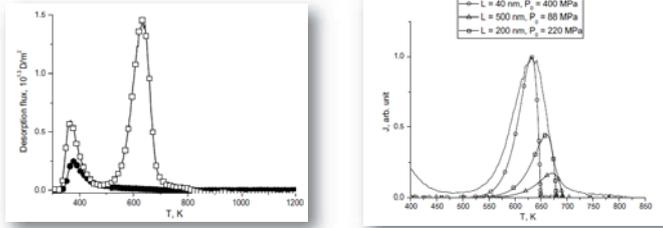


- Clear transformation at 1073-1273 K! (vacancy clusters transforms to bigger voids or single vacancies?)
- At the end of experiments small pores have been observed.



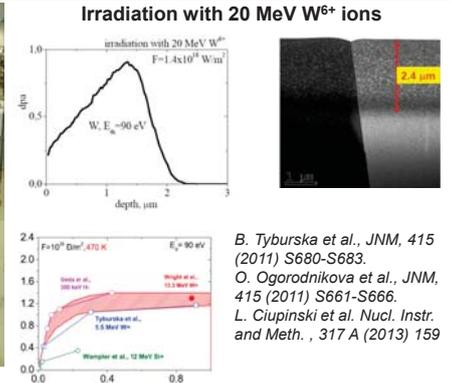
# Desorption from pores: Calculations

E.D. Marenkov et al. Nucl. Instr. and Meth. B 269 (2011) 876–880.



- The peak at 630 K correlated with the presence of pores in the sample.
- This peak can be described by deuterium gas release from pores distributed in the 500 nm thick surface layer with the pressure 90-400 MPa.

# Damage by MeV ions (IPP, Garching)

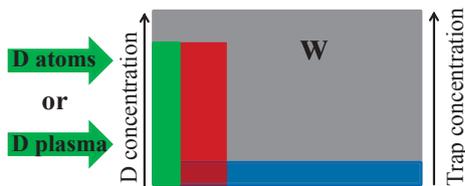


B. Tyburska et al., JNM, 415 (2011) S680-S683.  
O. Ogorodnikova et al., JNM, 415 (2011) S661-S666.  
L. Ciupinski et al. Nucl. Instr. and Meth. , 317 A (2013) 159

# Deuterium filling of traps

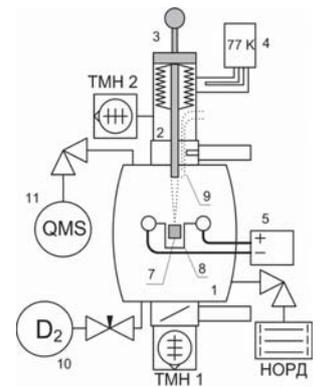
HABS (Ljubljana)	PlaQ (Garching) – ECR plasma
Average atomic energy: 0.2 eV/D	Ion energy: 20-200 eV/D
Atomic flux: $3.5 \cdot 10^{18}$ D/m <sup>2</sup> s	Ion flux: $(5-10) \cdot 10^{19}$ D/m <sup>2</sup> s
Temperature: 300-800K	Temperature: 300-800K

S. Markelj et al., JNM, 438 (2013) S1027-S1031.  
O. Ogorodnikova et al., JNM, 442 (2013) 518-527.



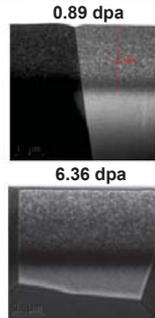
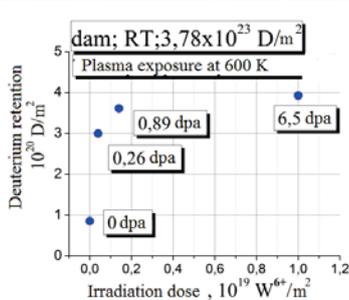
# UHV TDS Stand

- 1 – TDS chamber
- 2 – Sample exchange chamber
- 3 – Linear feed through
- 4 – Nitrogen trap
- 5 – Power supply
- 7 – Sample
- 8 – Heater
- 9 – Thermocouple
- 10 – Calibration system
- 11 – Quadrupole mass-spectrometer



- The base pressure is  $<5 \cdot 10^{-9}$  mbar
- Sample exchange chamber allows to reduce time of experiments

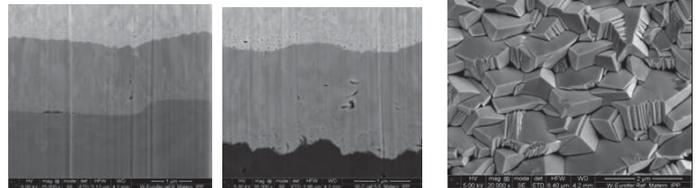
# Dependence on dpa level



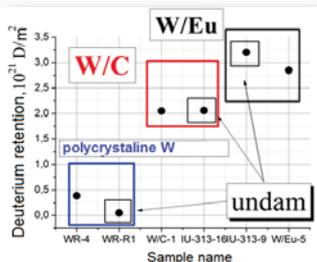
- D retention in damaged W has a trend to saturation at 0.5-1 dpa
- However slow gradual increase is observed till highest dpa, that is in agreement with TEM images

# Tungsten coatings

- W coatings on graphite and steel (Eurofer) substrates deposited using the method of combined magnetron sputtering and ion implantation – CMSII.
- Thickness - 10  $\mu$ m.



# W Coatings & Polycrystalline W

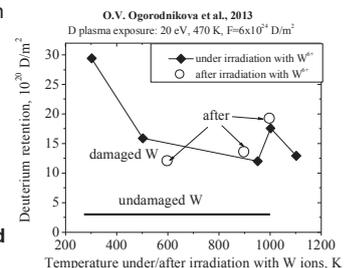


Fluence –  $2.2 \cdot 10^{25}$  D/m<sup>2</sup>  
Temperature – 600 K  
W coatings – 10  $\mu$ m  
Damaged zone – 2.5  $\mu$ m

- Influence of radiation damages on retention in W coatings is very small, due to presence of many virgin defects.
- Higher retention in W coatings is due to 4 times deeper zone of defects.
- Coatings on Eurofer accumulates more D due to diffusion in to the bulk of substrate.

# Annealing of radiation damage

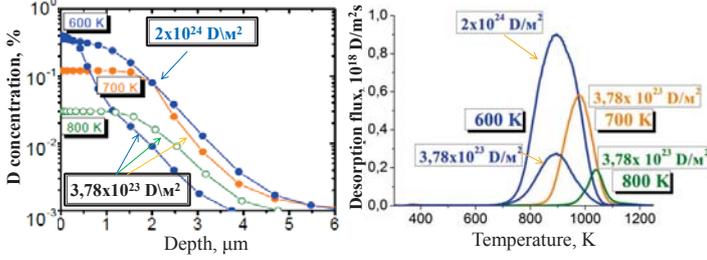
- ❖ The D retention reduces intensively in the temperature range between 300 and 700 K.
- ❖ Annealing up to 1100 K reduces only partly the density of radiation defects in W and, consequently, the D retention
- ❖ An increase of the D retention at around 1000 K correlates with agglomeration of radiation-induced defects in clusters observed by TEM (L. Ciupinski et al., 2013) .



# Atomic exposure

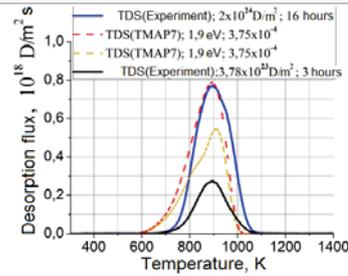


With atoms one can study interaction with original pattern of traps.



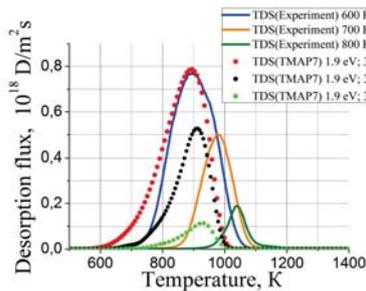
- High D retention as in the case of plasma!
- D concentration in traps decreases with the implantation temperature.
- At 700 K and 800 K traps are filled already at the fluence of  $3 \times 10^{23} \text{ D/m}^2$ .
- One can see the shift of the TDS peak with the increase of exposure temperature.

# Calculations: 600 K



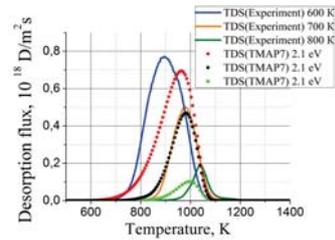
- Dynamics of traps filling depends on recombination coefficient at the surface.
- Even with the highest recombination coefficient the rate of traps filling is faster than in experiment  $\Rightarrow$  some factors reduce the transport rate

# Calculations: all spectra



- If one uses the same parameters for higher exposure temperature, the calculated width of peaks becomes more than experimental one
- Peak positions are also shifted

# Calculations: all spectra

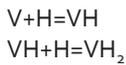


- To get a better agreement with the experiment at 700 K we used higher energy of traps and inhomogeneous profile
- Probably, defect structure changes with the increase of exposure temperature. Or we use a wrong model?

# Calculations: Traps with several places

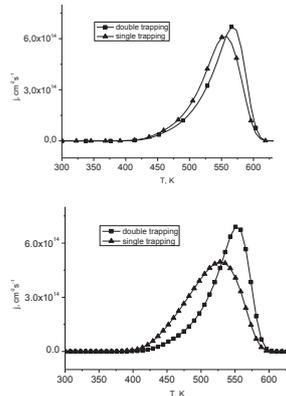


Real defects can trap several atoms. One can add to the model additional processes:



$$\frac{\partial C_s}{\partial t} = D \frac{\partial^2 C_s}{\partial x^2} - \frac{\partial}{\partial t} (C_{n1} + C_{n2})$$

First calculations confirm that this assumption makes peaks thinner!



# DFT calculations: Effect of strains on radiation defects in W



Dr. Maria Ganchenkova  
Material department, MEPhI

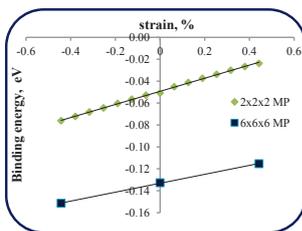
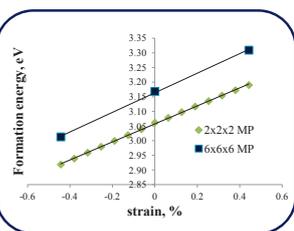
1. DFT based code VASP
2. PAW PBE
3. SCs 250 atoms
4.  $k$ -points  $2 \times 2 \times 2 - 8 \times 8 \times 8$  MP mesh
5. 400 eV of cut-off energy



# Vacancies in W: Pressure effect



# Summary



1. Strain conditions considered: isotropic 3D strain within  $\pm 0.44\%$
2. Energies change linearly within the considered interval of strain values
3. Formation energy varies within  $\pm 0.15 \text{ eV}$
4. Binding energy varies within  $\pm 0.02 \text{ eV}$

- Radiation damages lead to accumulation of high amount of hydrogen isotopes (up to  $\sim 1$  at.%).
- Different W materials has similar maximum concentration of traps. In the case of high initial trap concentration the effect of radiation on retention is smaller.
- Both keV and MeV ion irradiation produce a high concentration of defects with the high detrapping energy (1.8-2.2 eV), which determine retention at high temperatures. These defects are, very likely, vacancy clusters.
- At 1000-1200 K vacancy clusters has transformation to bigger size clusters (voids). Large scale pores can be formed which are stable at very high temperatures.
- The filling rate of defects depends strongly on impurities on the surface and in the incident flux.